



***A Roadmap to the
realization of fusion energy
(see www.efda.org)***

***Working Group on Energy (AKE)
Dresden***

***Acknowledgments: P. Barabaschi, D. Borba,
G. Federici, L. Horton, R. Neu, D. Stork, H. Zohm***

***Francesco Romanelli
European Fusion Development Agreement
EFDA Leader and JET Leader
4 March 2013***



Energy challenge for Europe

Sustainability

Security of supply

Economic competitiveness

Fusion Energy

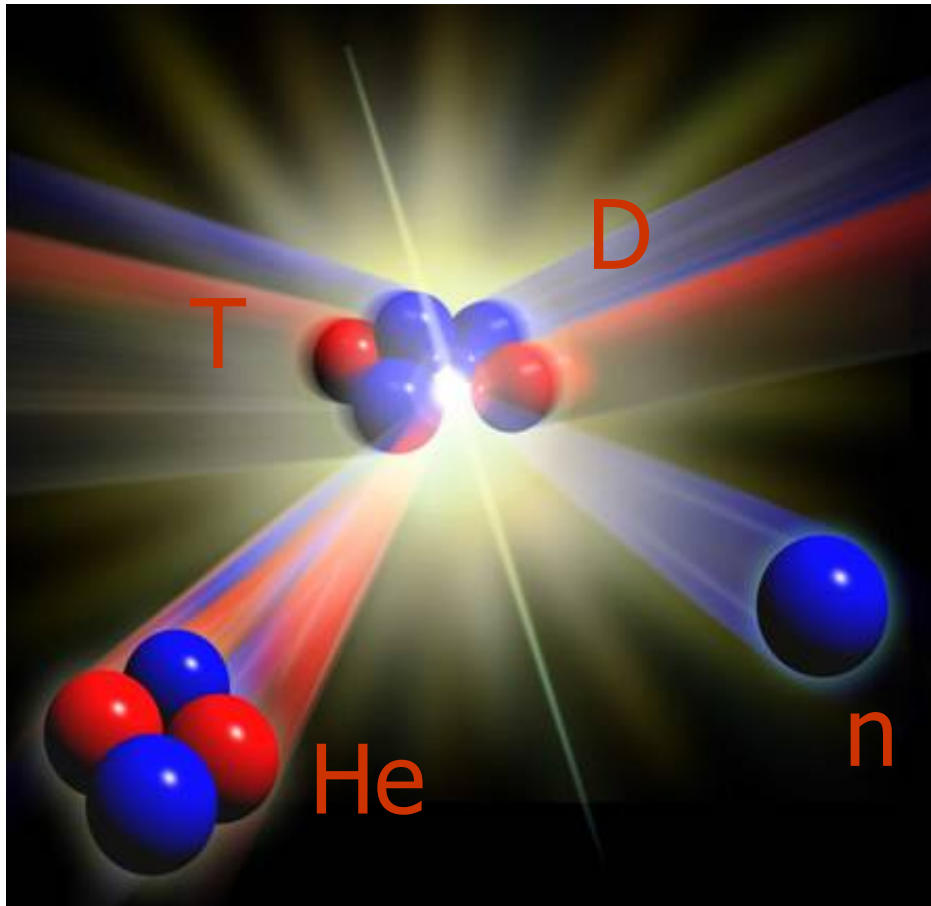
Unlimited and diffuse energy source

No greenhouse gases

Intrinsically safe

Environmentally responsible

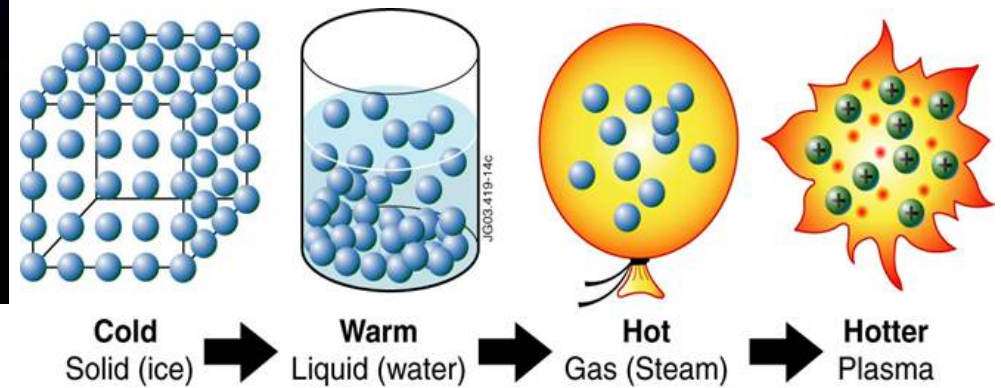
How to make fusion?



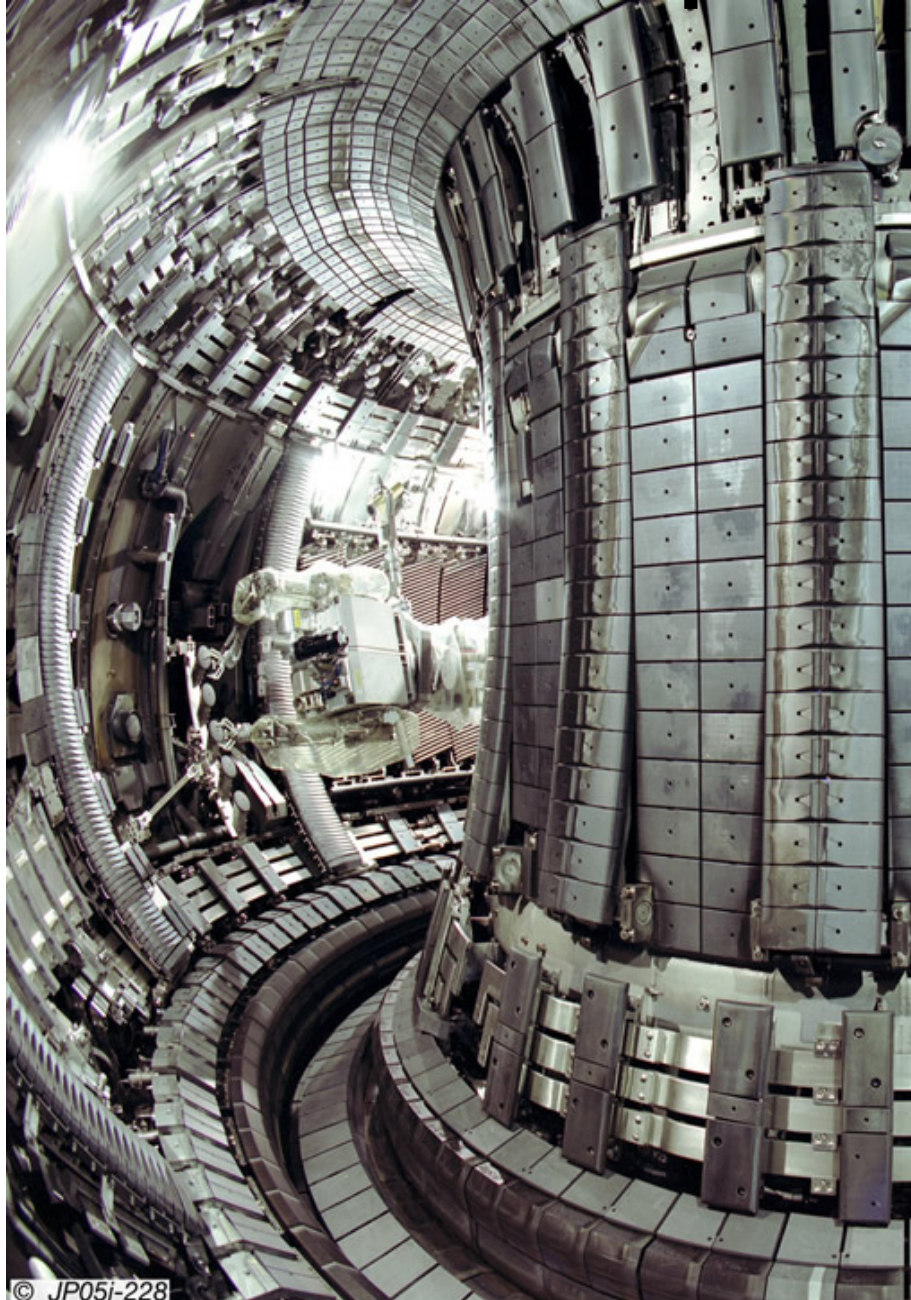
Reacting nuclei are charged
⇒ they repel each other

Heat nuclei up to 200 Million °C

Matter is in the *plasma* state




The Joint European Torus (JET)



Fusion power has been produced on JET



25MW of auxiliary power to heat the plasma



The challenge of confining a hot plasma is achieved!

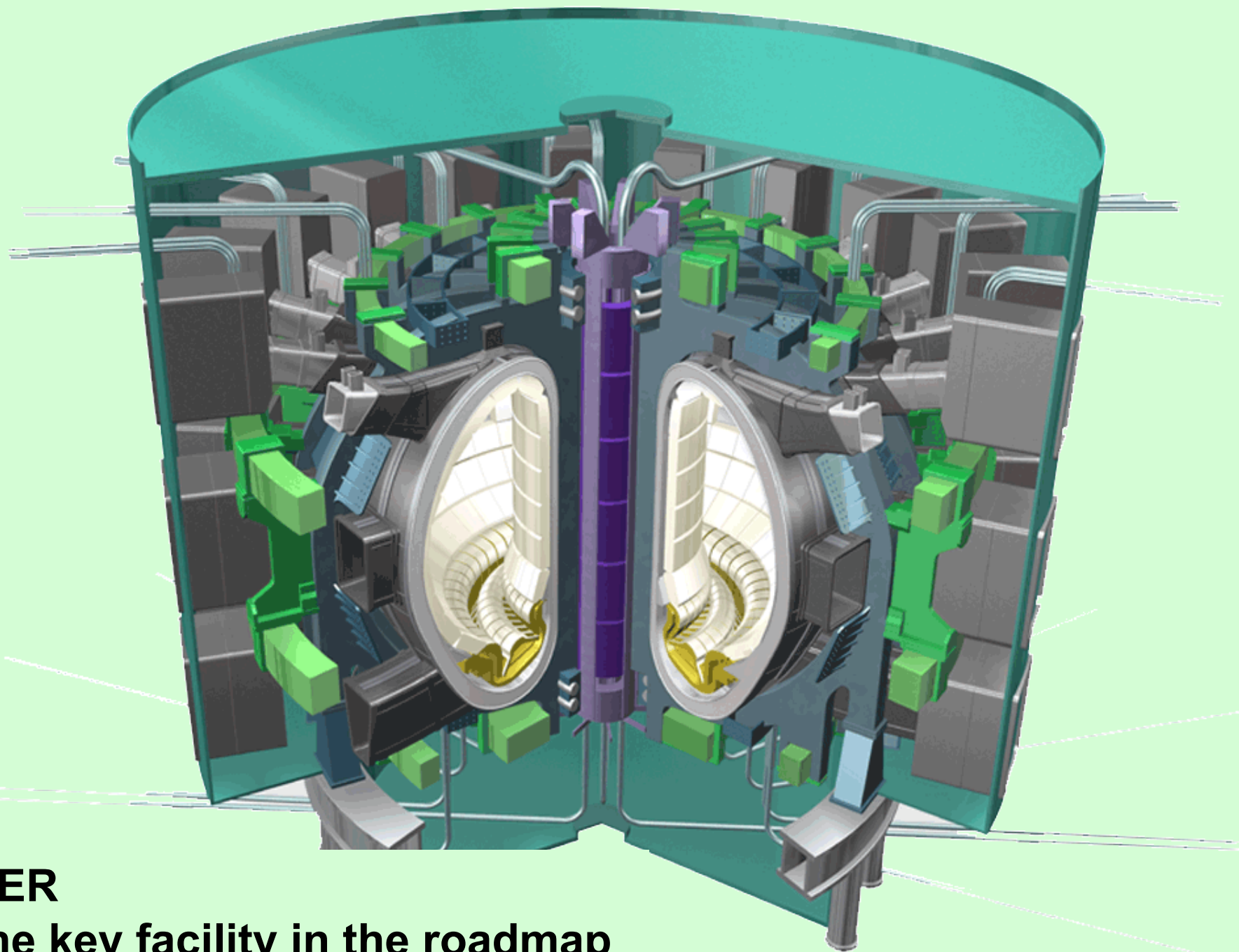
What do we need to make a power plant?

European Commission proposal for Horizon 2020 states the need of *an ambitious yet realistic roadmap to fusion electricity by 2050.*

→ Require **DEMO** construction in ~ 2030

The present roadmap

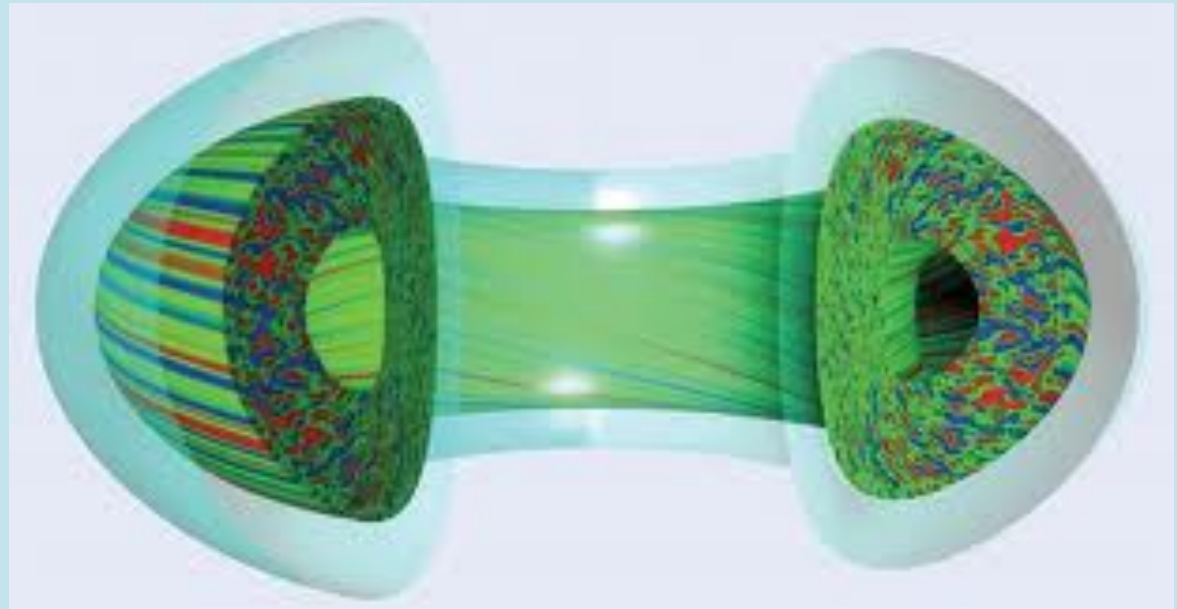
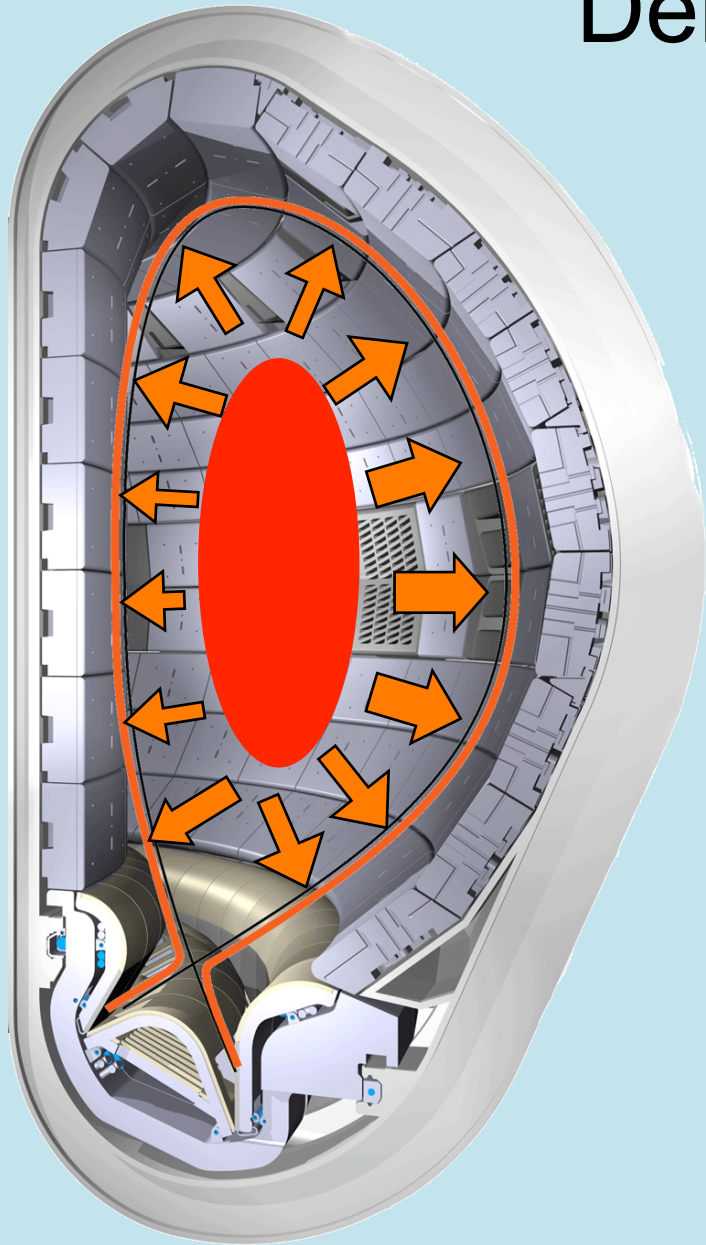
- Pragmatic approach to fusion energy.
- Focus the effort of European laboratories around **8 Missions**
- Ensure innovation through early industrial involvement
- Exploit the opportunities arising from international collaborations



ITER
The key facility in the roadmap

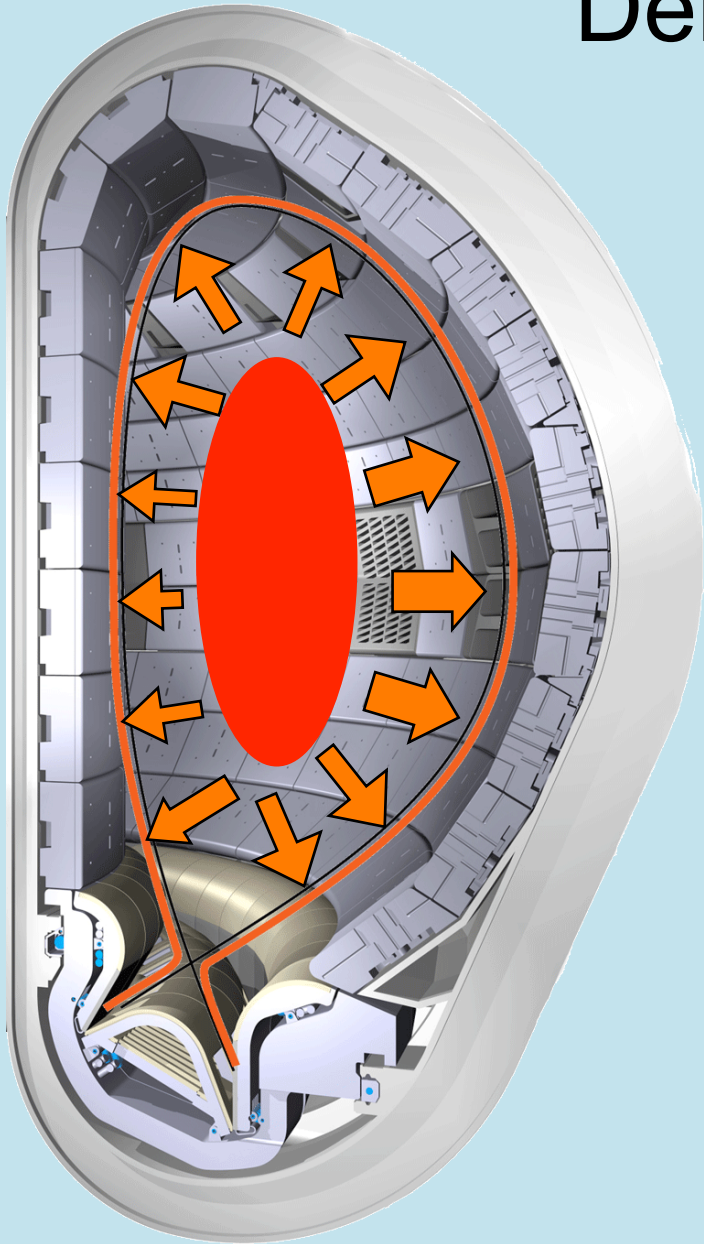
Mission 1: Plasma regimes for a reactor

Demonstrate a net energy gain



Mission 1: Plasma regimes for a reactor

Demonstrate a net energy gain

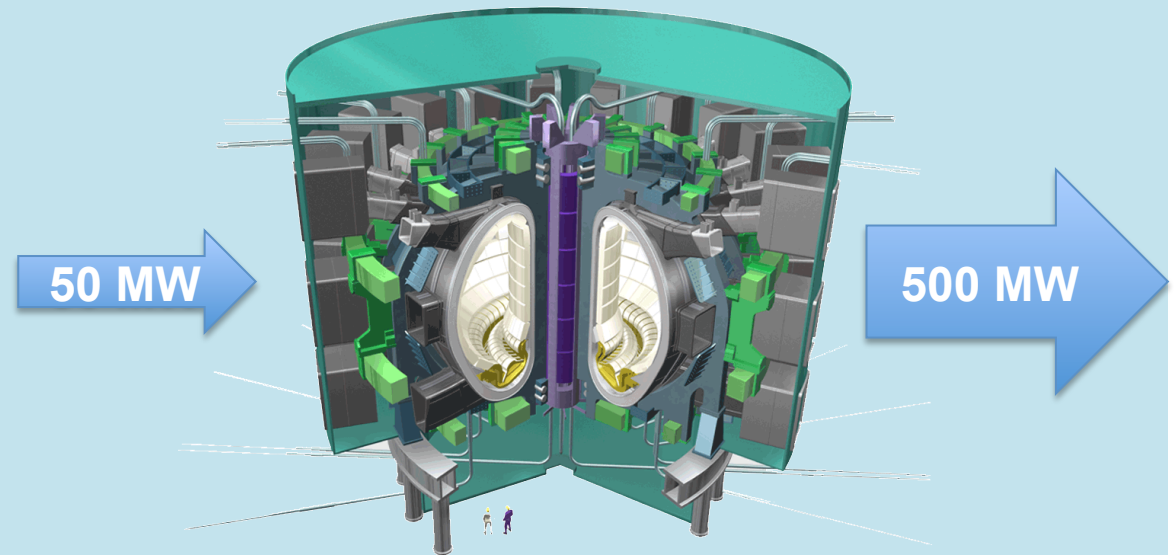
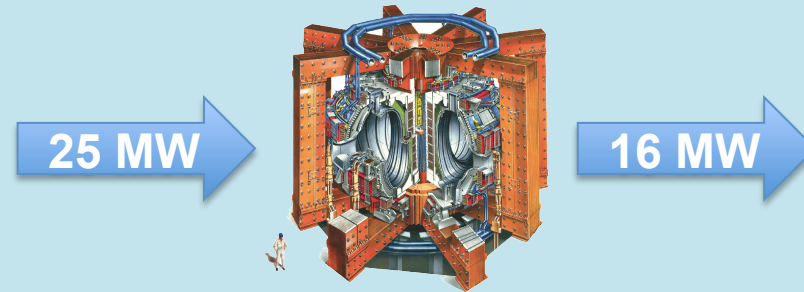
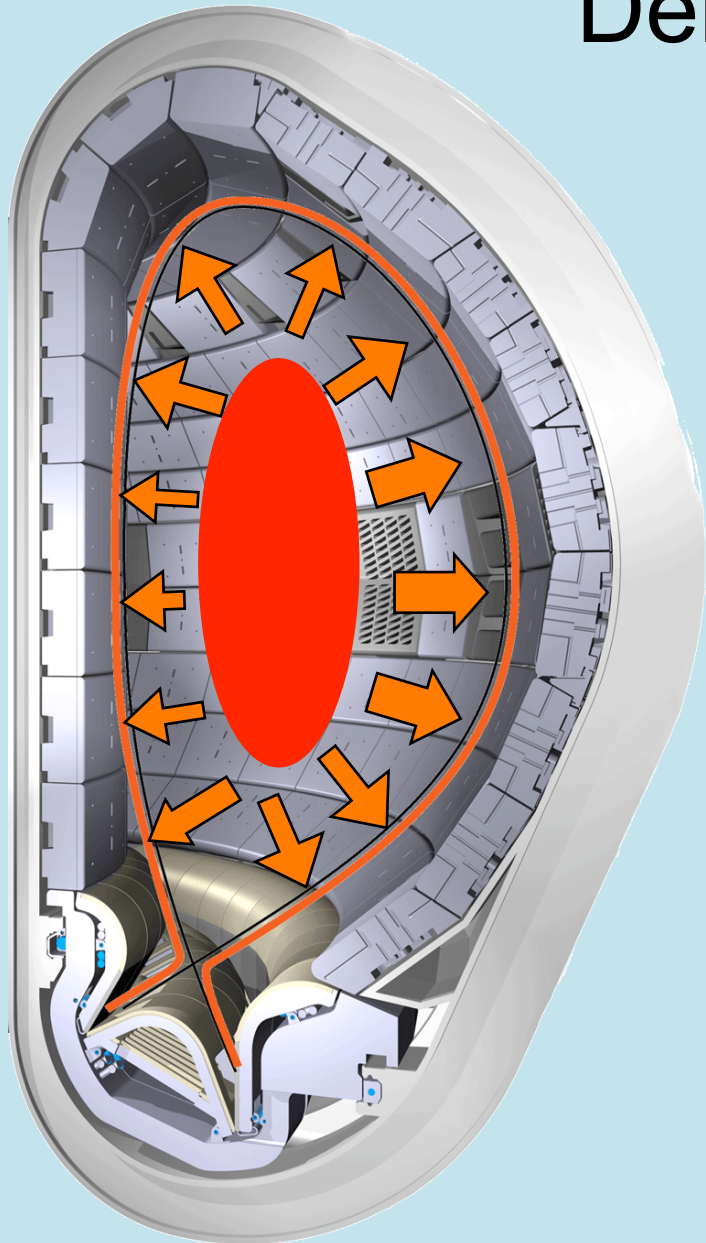


- Energy losses increase at most as the radius R of the device
- Fusion power increases as the volume ($\approx R^3$)

MAKE LARGER DEVICES

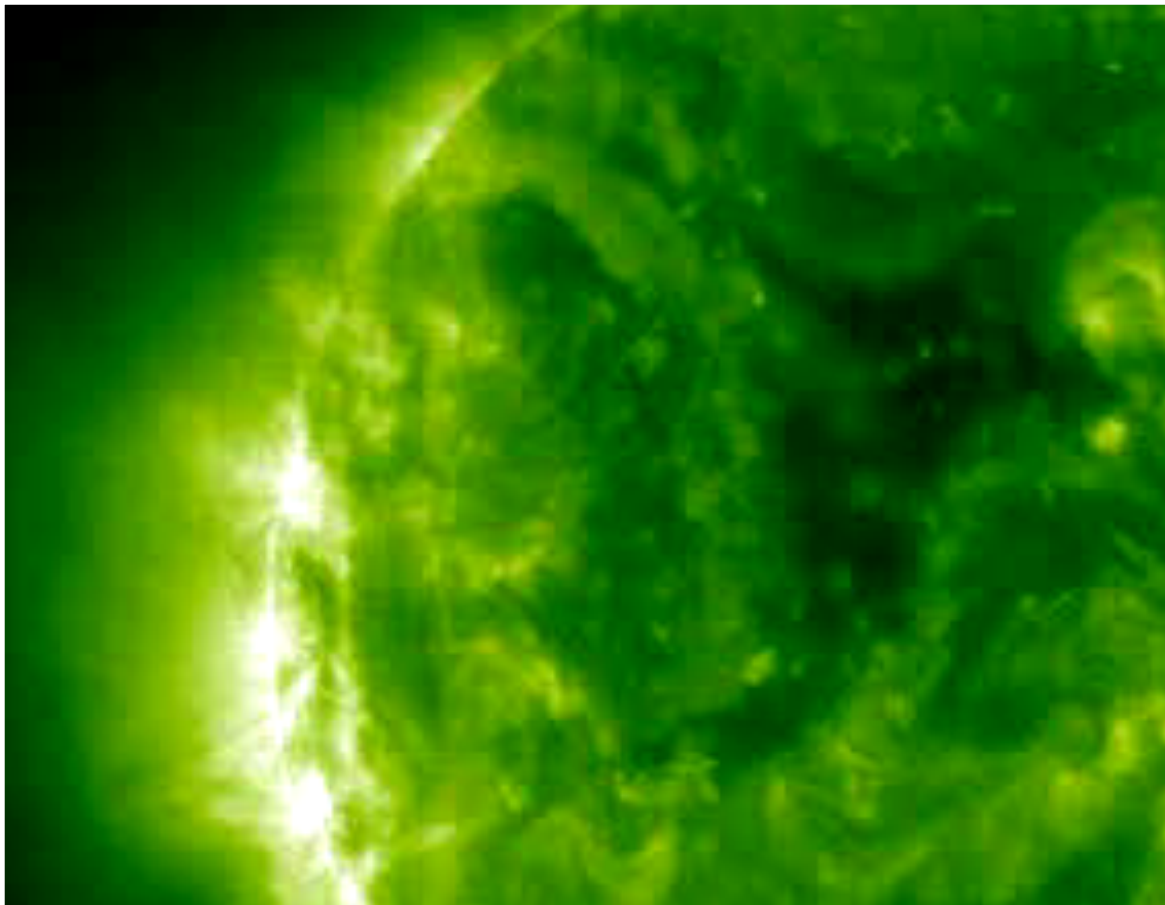
Mission 1: Plasma regimes for a reactor

Demonstrate a net energy gain



Mission 1: Plasma regimes for a reactor

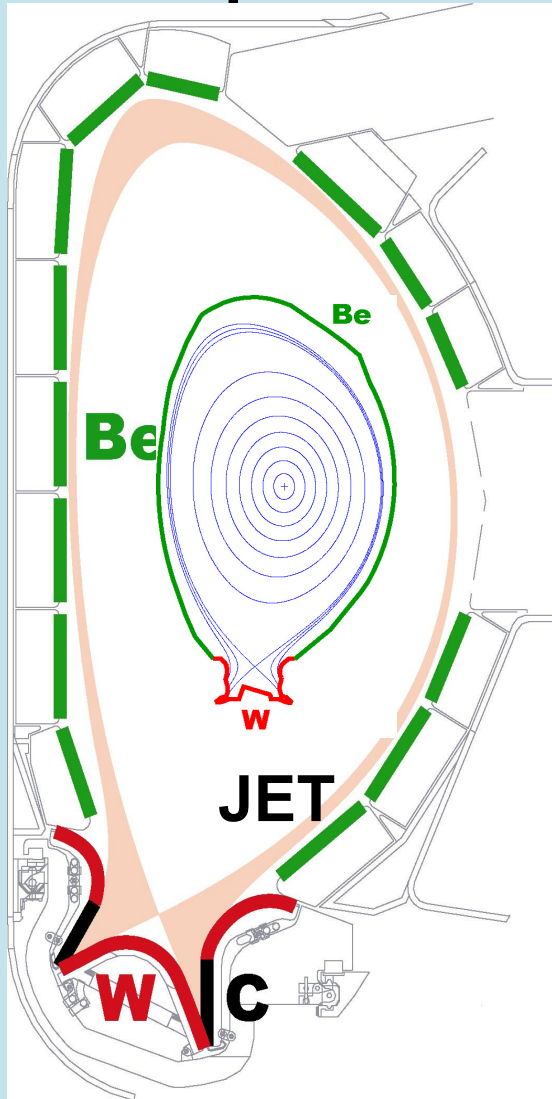
Control plasma instabilities



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Mission 1: Plasma regimes for a reactor

Compatibility between plasma and wall materials



ITER

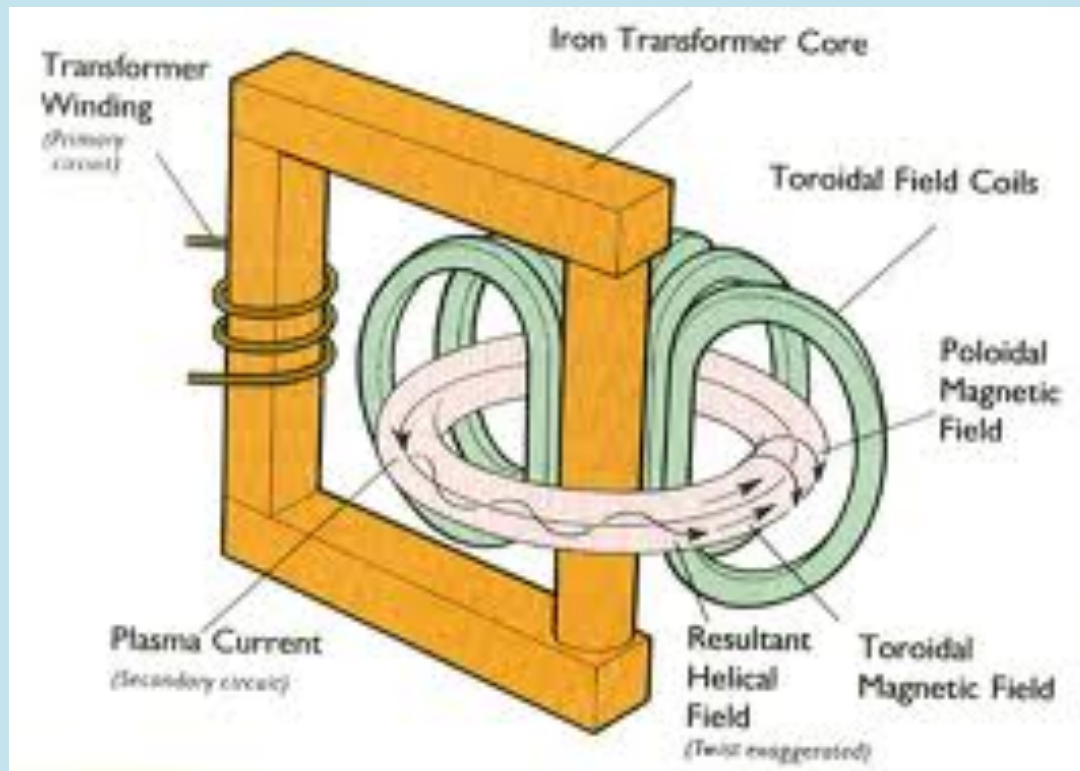


ASDEX U

Tungsten foreseen for a reactor
to minimize erosion

Mission 1: Plasma regimes for a reactor

Steady state regimes



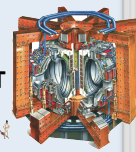
At high plasma pressure most of the current can be generated by the plasma itself (bootstrap mechanism)

These regimes are still under development in tokamaks and will be demonstrated in ITER beyond 2030.

The Roadmap in a nutshell

- 1. Plasma operation
- 2. Heat exhaust
- 3. Materials
- 4. Tritium breeding
- 5. Safety
- 6. DEMO
- 7. Low cost
- 8. Stellarator

JET

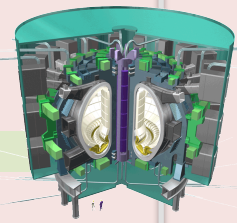


Inductive
Steady state

European Medium Size Tokamaks
+ International Collaborators



JT60-SA



DEMO decision

2010

2020

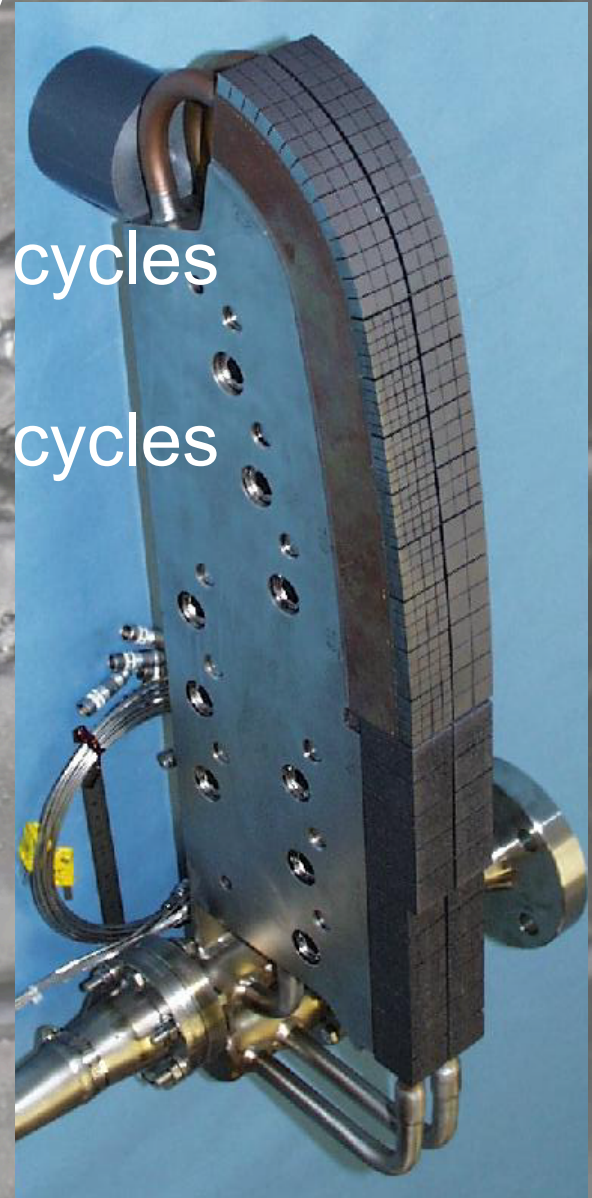
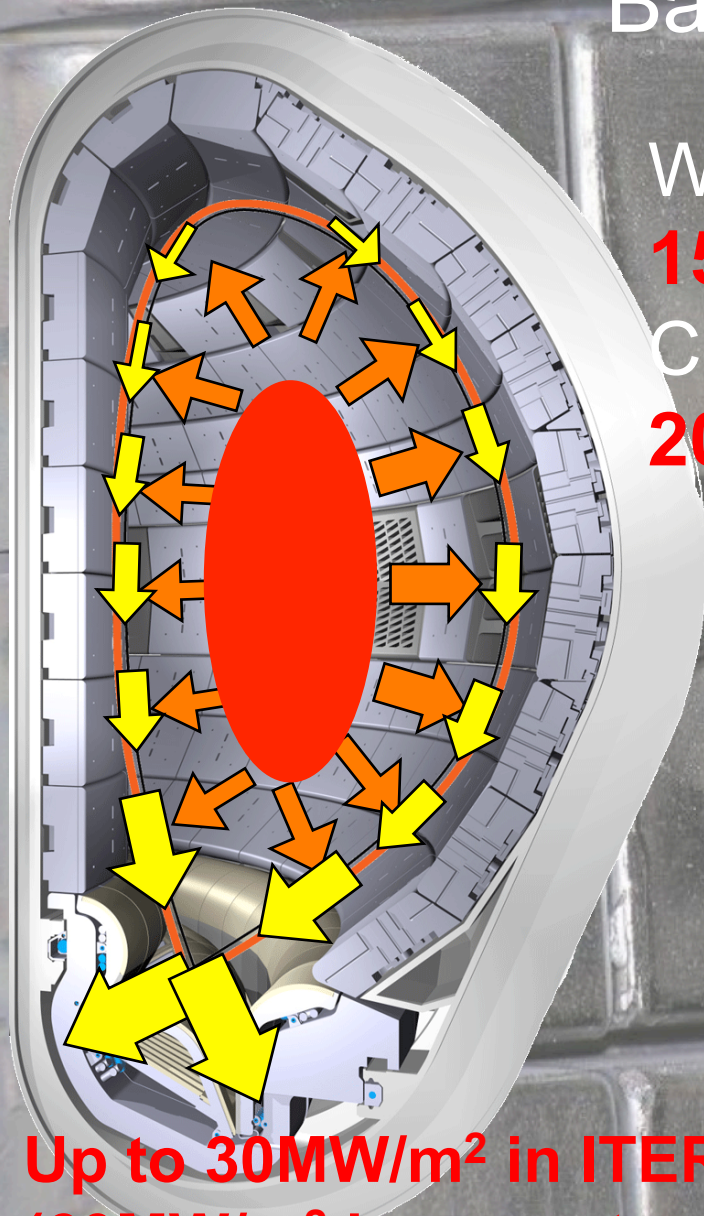
2030

2040

2050

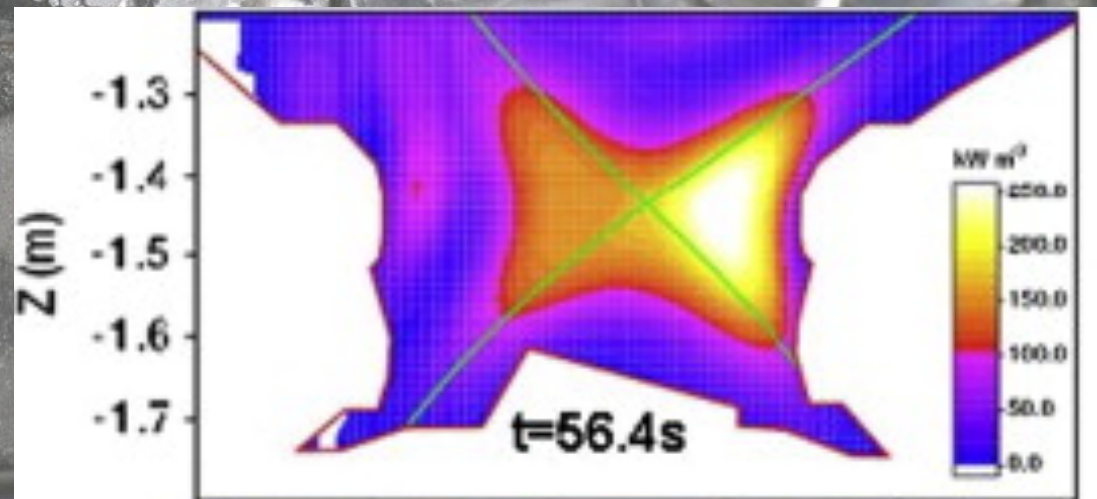
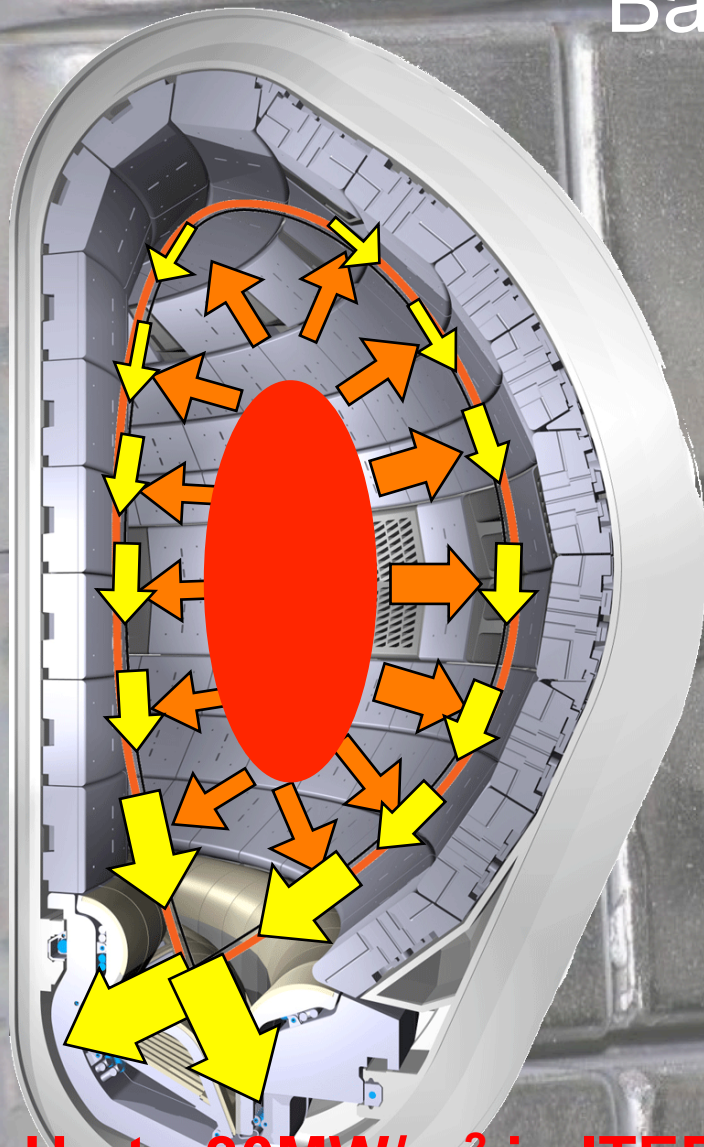
Mission 2: Heat and particle exhaust Baseline strategy

W macrobrush:
15 MW/m² x 1000 cycles
CFC monoblock
20 MW/m² x 2000 cycles



**Up to 30MW/m² in ITER
(60MW/m² in a reactor ~ heat flux on the surface of the Sun!)**

Mission 2: Heat and particle exhaust Baseline strategy



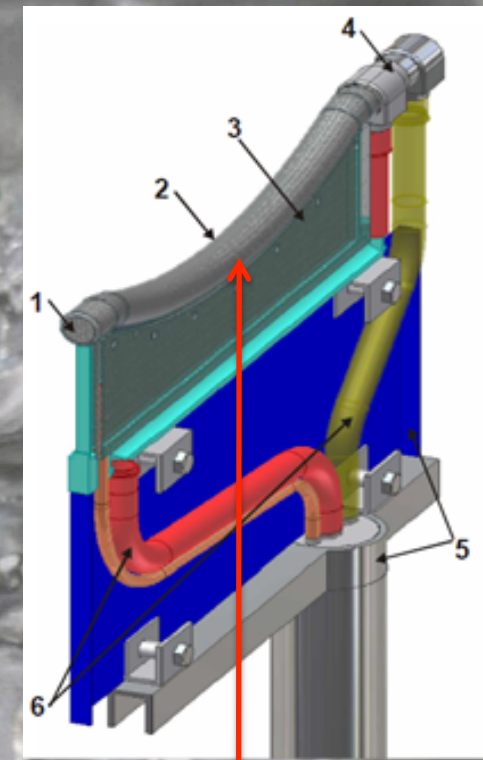
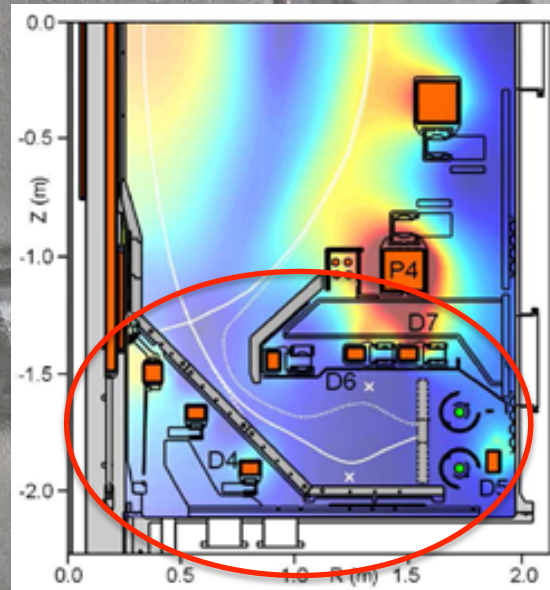
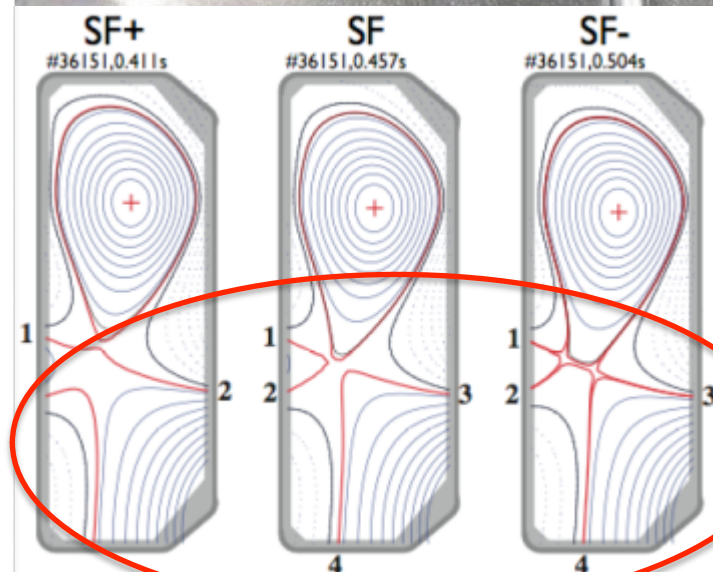
Divertor detachment

Up to 30MW/m^2 in ITER

(60MW/m^2 in a reactor ~ heat flux on the surface of the Sun!)

Mission 2: Heat and particle exhaust

Alternative strategies



Liquid-metals

- Proof-of-principle on medium size experiments
- Assess reactor-relevance in parallel

The Roadmap in a nutshell

1. Plasma operation

Inductive
Steady state

European Medium Size Tokamaks
+ International Collaborators

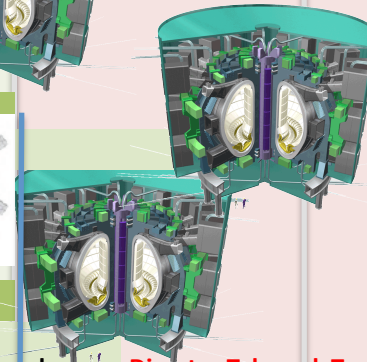


2. Heat exhaust

Baseline strategy

Advanced configuration and materials

European Medium Size Tokamaks + linear plasma + Divertor Tokamak Test Facility + International Collaborators Tokamaks



3. Materials

4. Tritium breeding

5. Safety

DEMO decision

6. DEMO

7. Low cost

8. Stellarator

2010

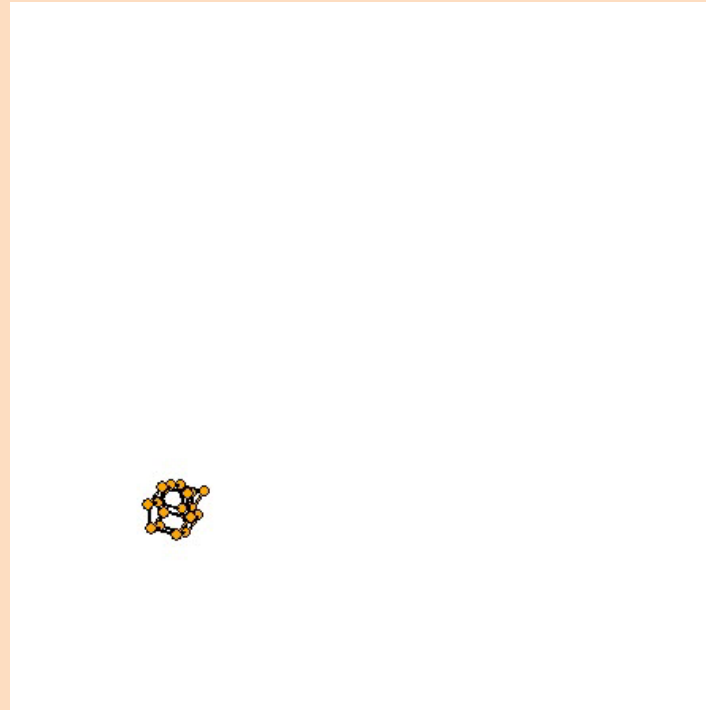
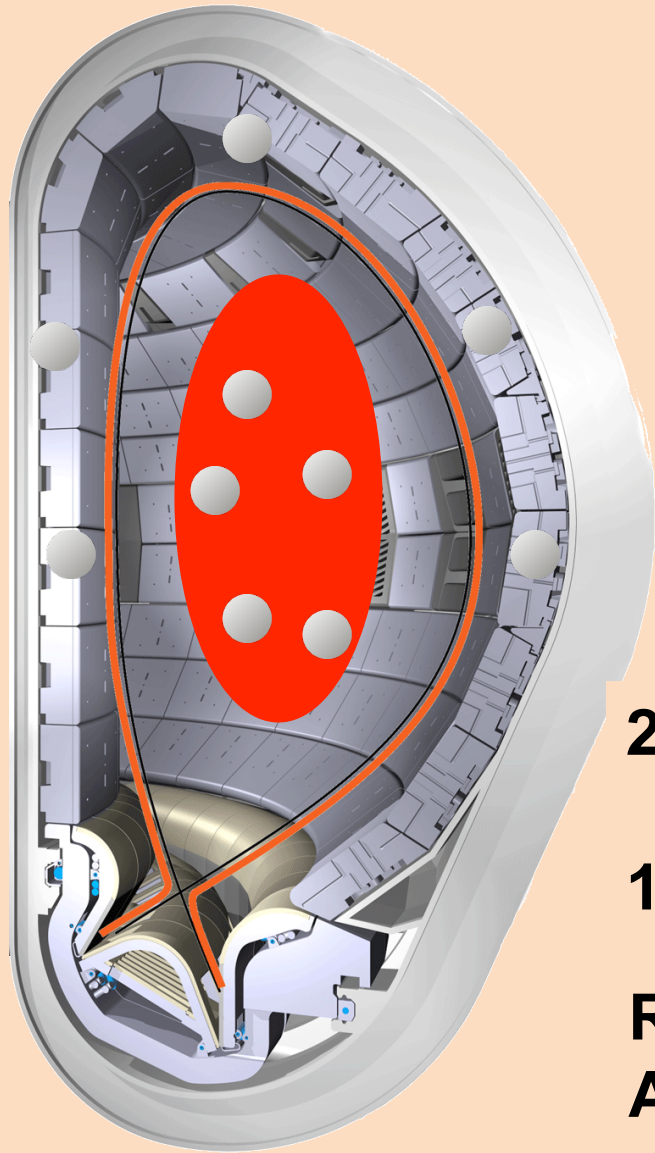
2020

2030

2040

2050

Mission 3: Develop neutron resistant materials



S. Dudarev

2 displacements per atom (dpa) in ITER

150 dpa in a fusion plant

Reduction of structural properties

Activation

D. Stork et al.

Material Assessment Report

Not a problem for ITER but must be solved for a reactor!

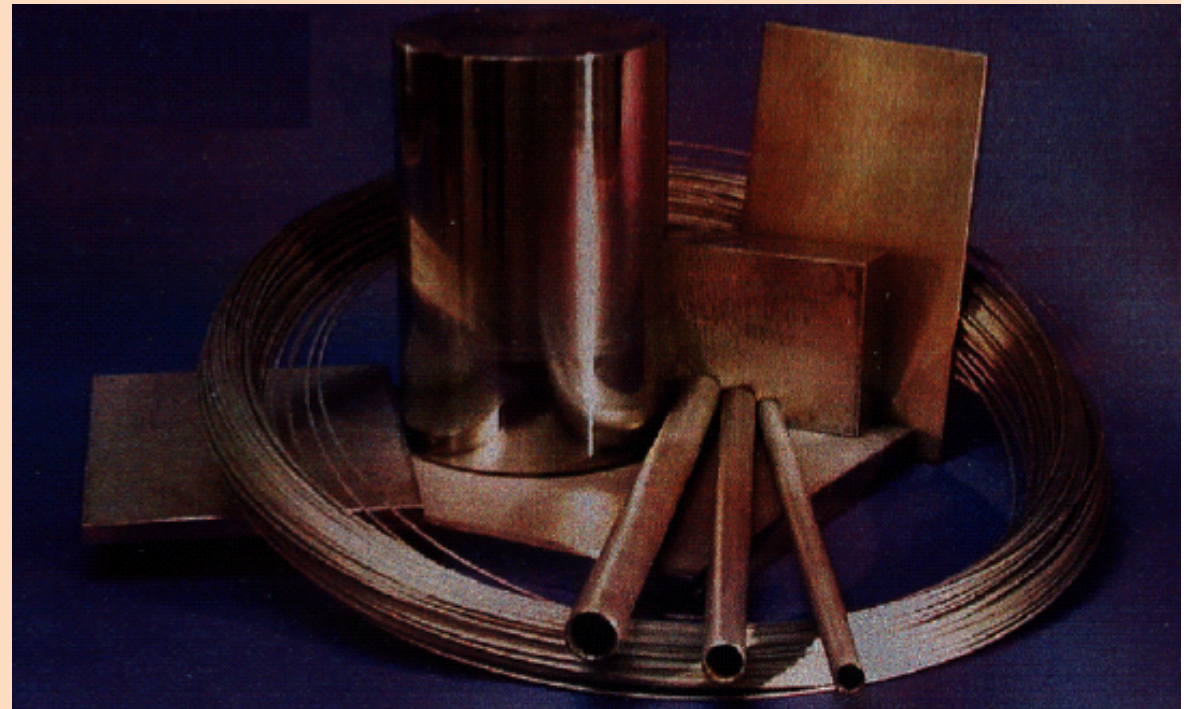
Mission 3: Develop neutron resistant materials

Existing candidate:

Low activation EUROFER

Selected range of temperature (300/550°C)

Tested in fission reactors up to 60 dpa



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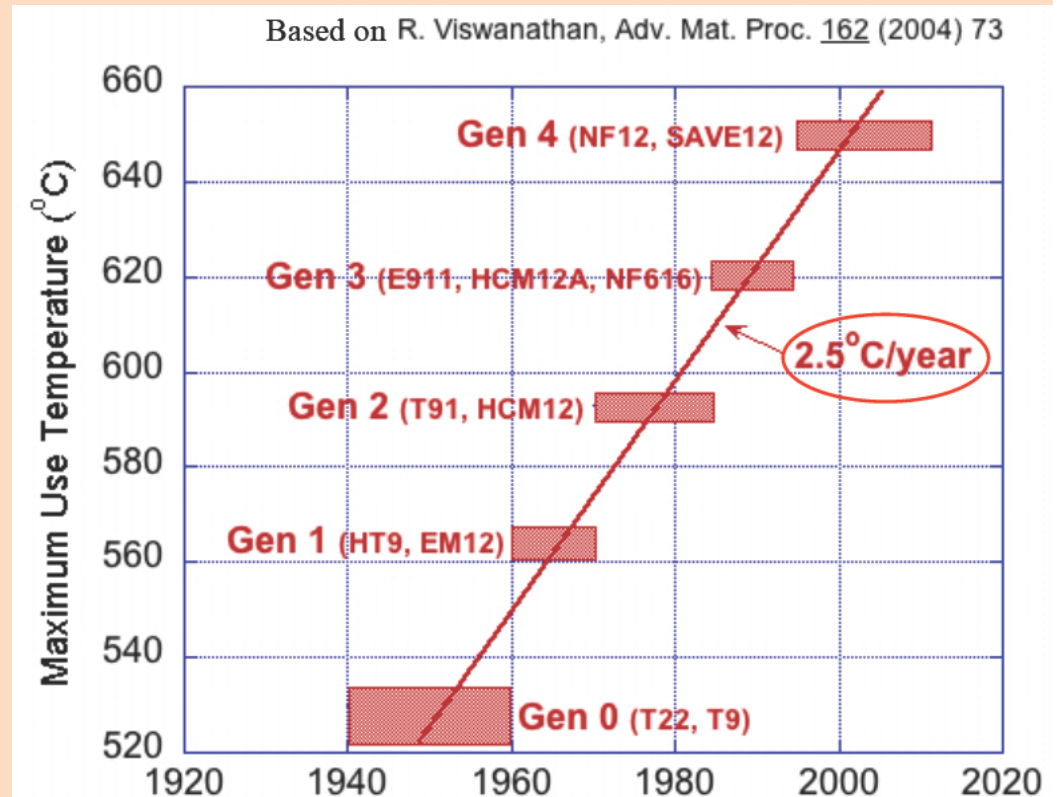
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Advanced materials under examination

ODS steels (650°C)

High-Temperature
Ferritic-Martensitic
steels



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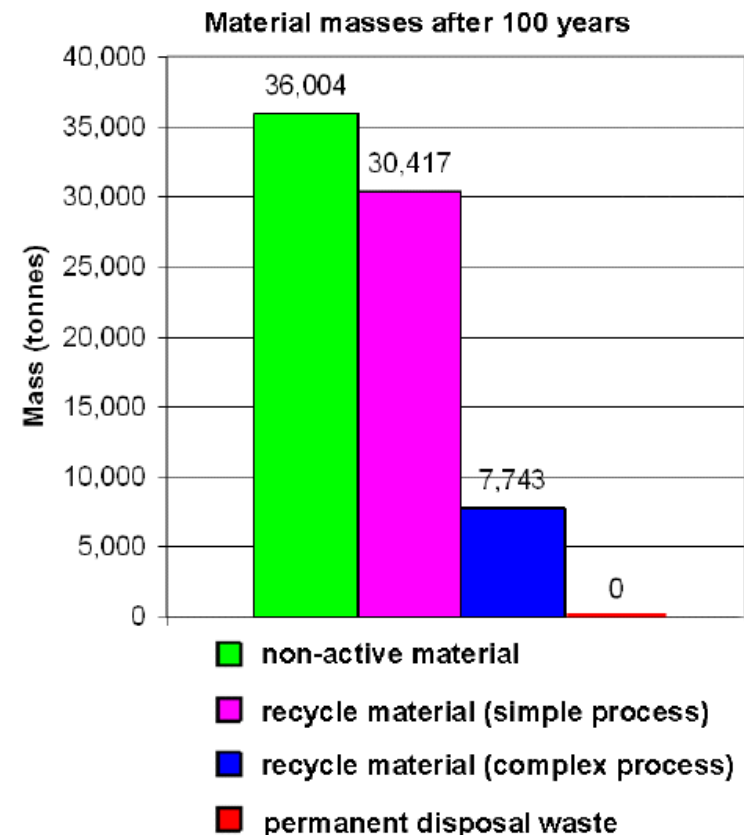
ODS steels (650°C)

High-Temperature

**Ferritic-Martensitic
steels**

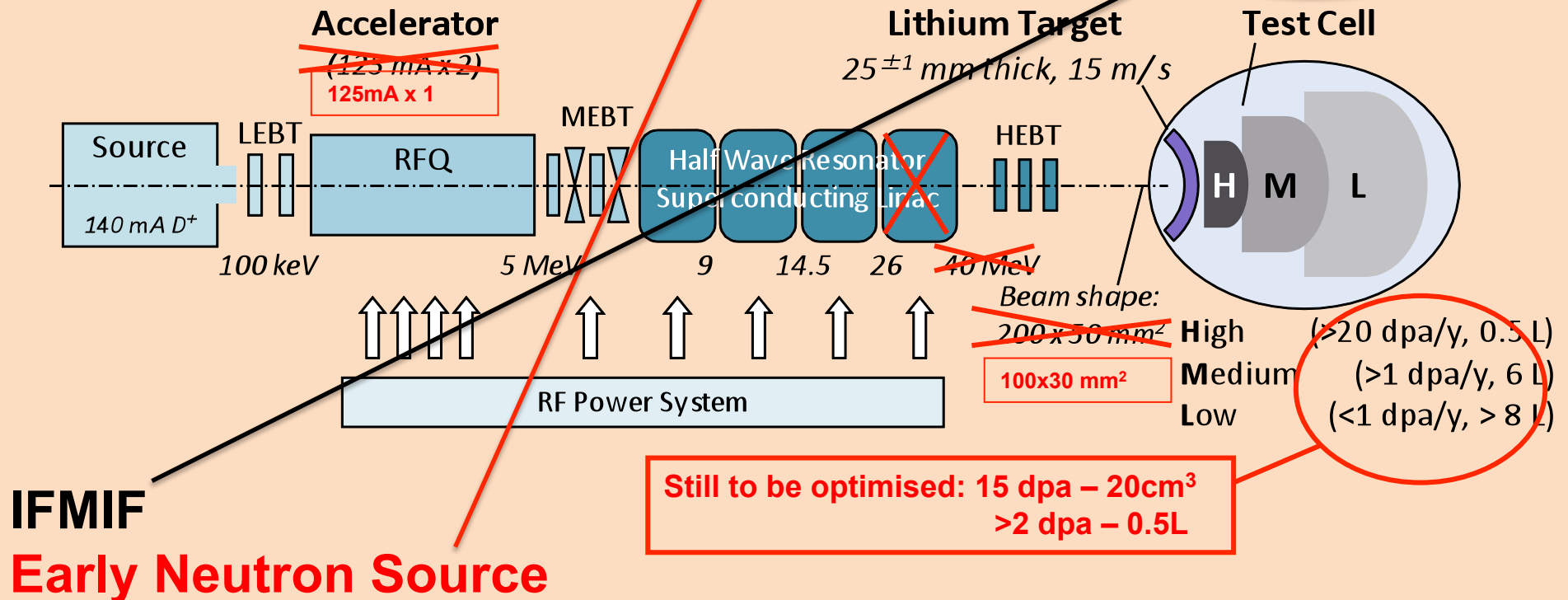
**Activation falls 10000 times
after 100 years**

**No need for permanent
waste repository**



Mission 3: Develop neutron resistant materials

	Onset of 14MeV effects	Calibration of 14MeV effects	Full database for the full exposure
DEMO Phase1	20dpa (Fe) 250-350°C 20cc	20dpa (Fe) 250-550°C 70cc	20dpa (Fe) 250-550°C 300cc
DEMO Phase2	50dpa (Fe) 250-350°C 20cc	50dpa (Fe) 250-550°C 70cc	50dpa (Fe) 250-550°C 300cc
Reactor		100dpa (Fe) 250-1200°C 70cc	100dpa (Fe) 250-1200°C 300cc



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1. Plasma operation

Inductive
Steady state

European Medium Size Tokamaks
+ International Collaborators

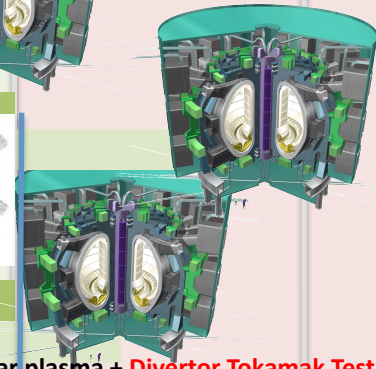


2. Heat exhaust

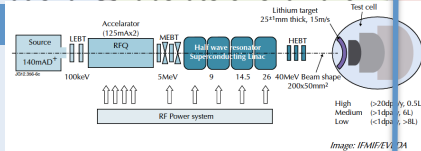
Baseline strategy

Advanced configuration and materials

European Medium Size Tokamaks + linear plasma + Divertor Tokamak Test Facility + International Collaborators Tokamaks



3. Materials



4. Tritium breeding

DEMO decision

5. Safety

6. DEMO

7. Low cost

8. Stellarator

2010

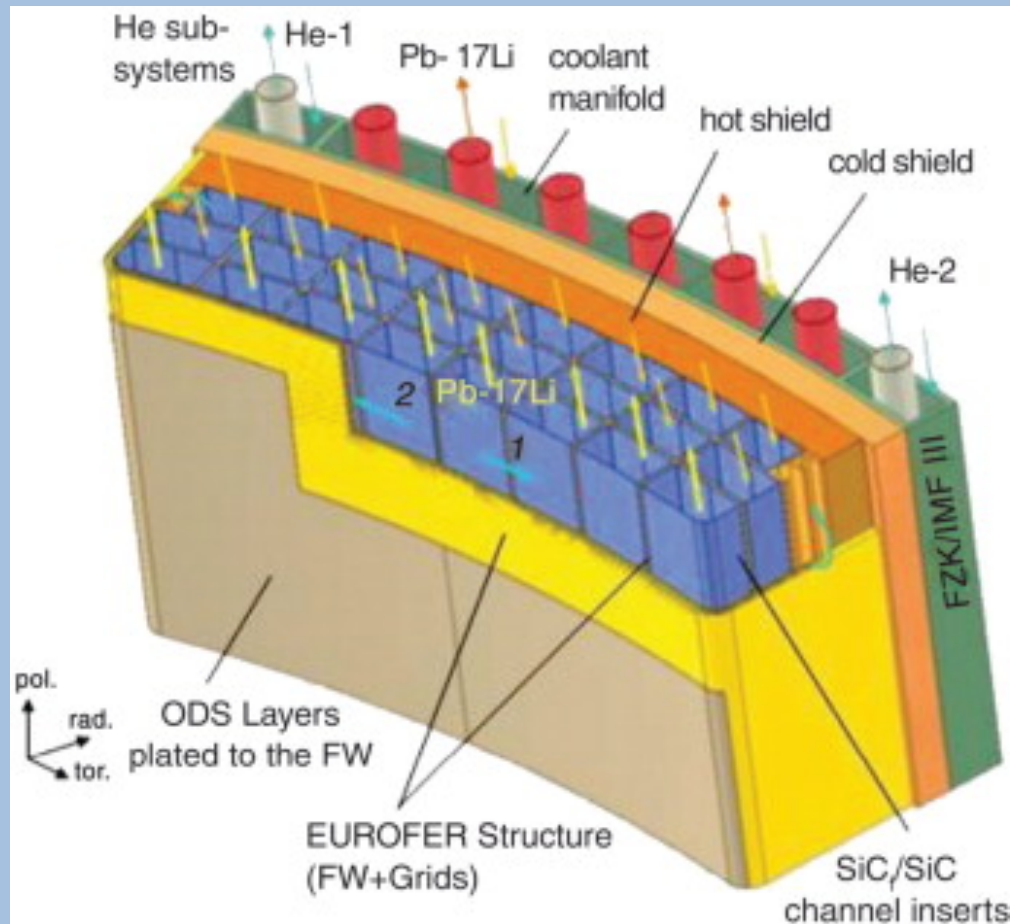
2020

2030

2040

2050

Mission 4: Tritium breeding



**A 1.5GWe reactor uses
~0.5kg Tritium/day**

Breeder

solid

liquid

Coolant

water

helium

self-cooled

Multiplier

Be

Pb

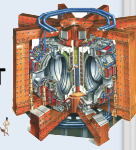
Efficient T extraction

**Efficient electricity
generation (balance
of plant)**

The Roadmap in a nutshell

1. Plasma operation

JET



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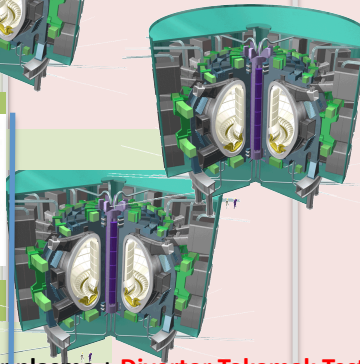
JT60-SA

2. Heat exhaust

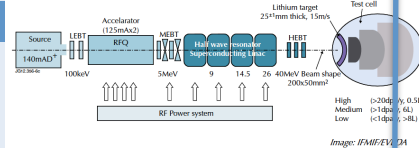
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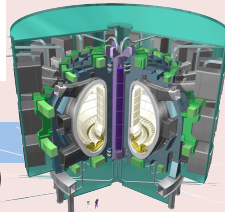
3. Materials



4. Tritium breeding

ITER Test blanket programme
Parallel Blanket Concepts

CFETR (CN)
FNSF (US)



DEMO decision

5. Safety

6. DEMO

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8. Stellarator

2010

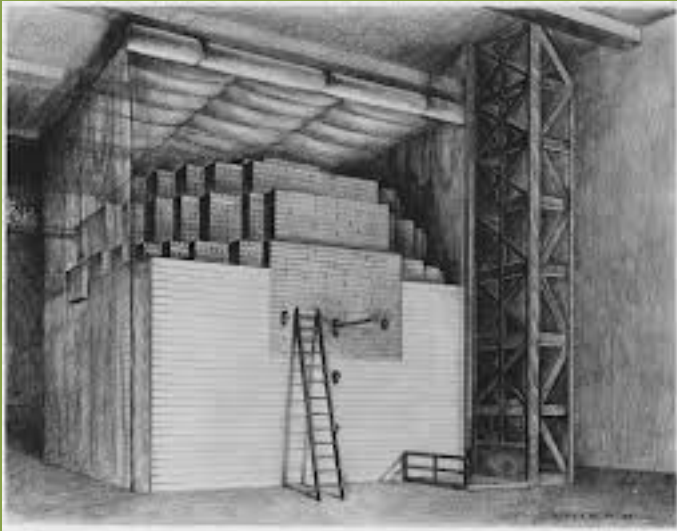
2020

2030

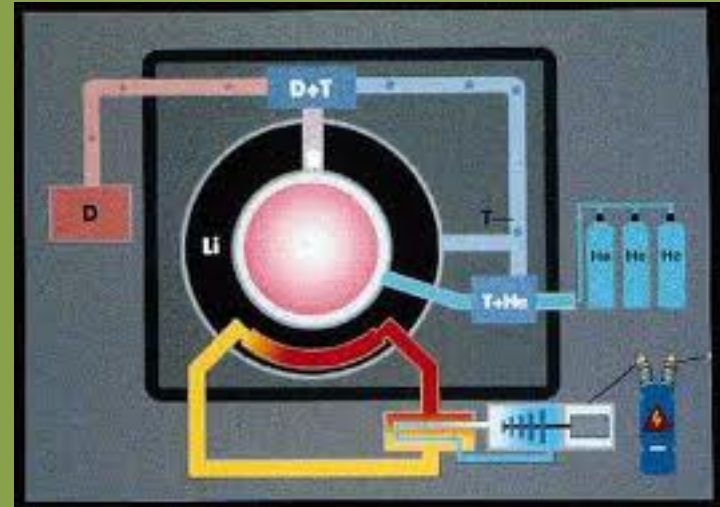
2040

2050

Mission 5: Implementation of inherent fusion safety features in DEMO design

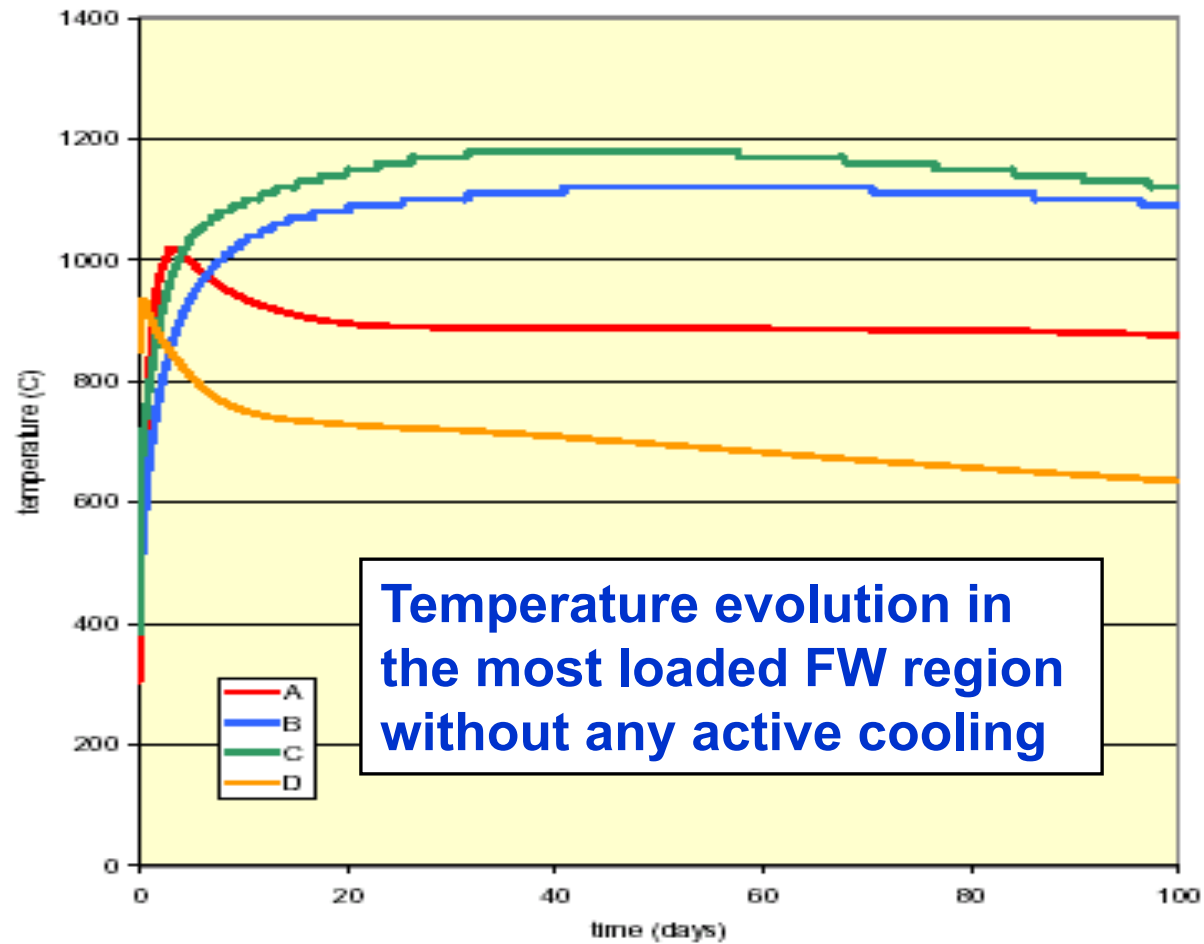


Fermi pile



Fusion reactor

Mission 5: Implementation of inherent fusion safety features in DEMO design



Mission 6: DEMO design

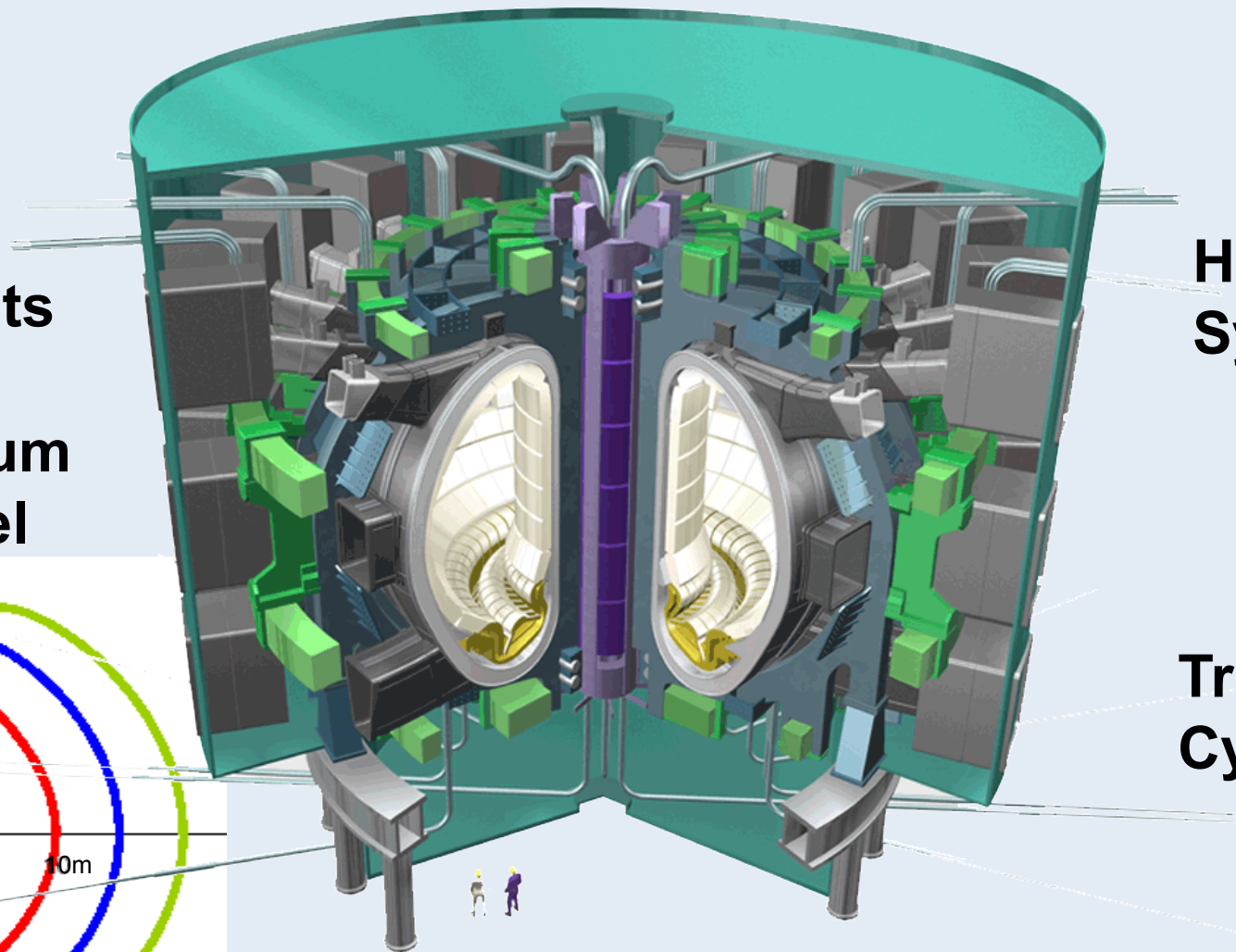
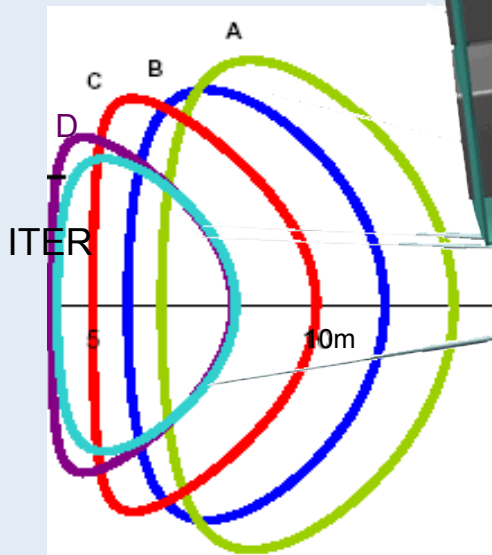
Balance of Plant

Magnets

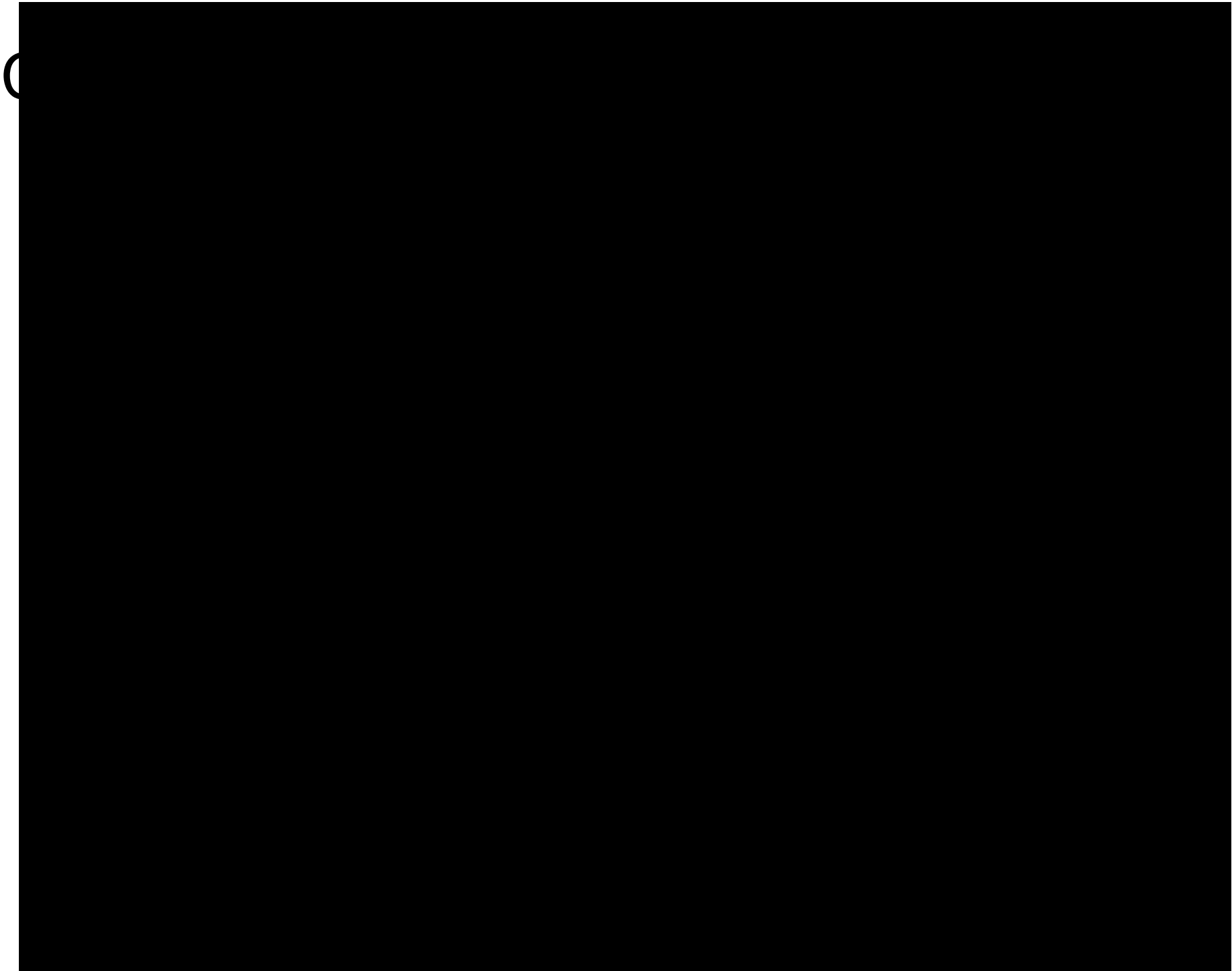
Vacuum Vessel

Heating Systems

Tritium Cycle

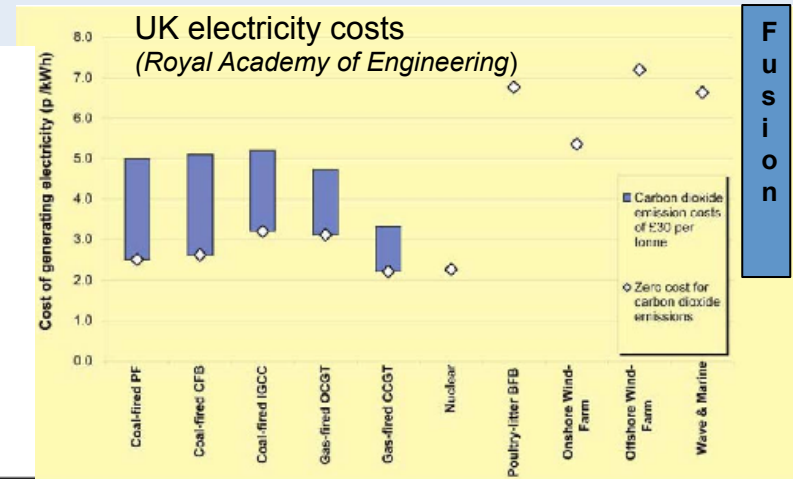
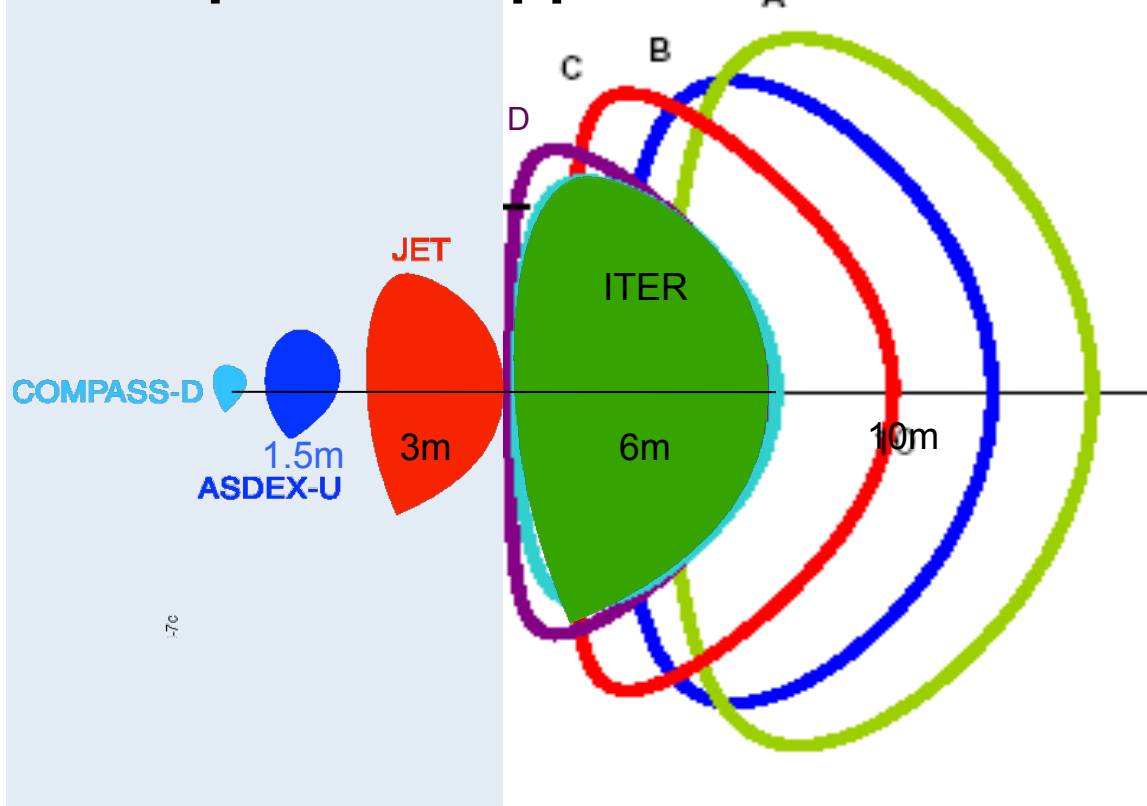


Remote Handling



Mission 7: Low cost of electricity

Step ladder approach



Cost of electricity from fusion expected to be competitive with other sources (IEA Levelised Cost Approach)

ITER is a moderate extrapolation from JET (x2)

The Power Plant (1.5GWe) expected to be a moderate extrapolation from ITER (x1-1.5) depending on the assumptions on physics and technology solutions (A=conservative; D=advanced)

EFDA Power Plant Conceptual Study

The Roadmap in a nutshell

1. Plasma operation

Inductive
Steady state

European Medium Size Tokamaks
+ International Collaborators



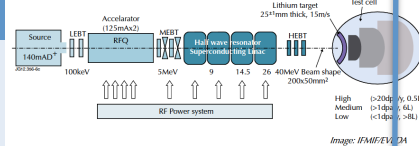
2. Heat exhaust

Baseline strategy

Advanced configuration and materials

European Medium Size Tokamaks + linear plasma + Divertor Tokamak Test Facility + International Collaborators Tokamaks

3. Materials



4. Tritium breeding

ITER Test blanket programme

Parallel Blanket Concepts

CFETR (CN)
FNSF (US)

5. Safety

Primary safety boundary the vacuum vessel (ITER approach)

Tritium management: define appropriate detritiation techniques and disposal routes

6. DEMO

CDA +EDA

DEMO decision

Construction

Reduced activation features expected to be incorporated already for the first set of DEMO components.

7. Low cost

Low capital cost and long term technologies

8. Stellarator

2010

2020

2030

2040

2050

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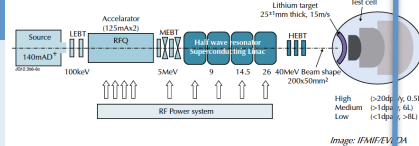
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ITER Test blanket programme

Parallel Blanket Concepts

CFETR (CN)
FNSF (US)

5. Safety

DEMO decision

6. DEMO

CDA +EDA

Construction

- Targeted R&D on
- Magnets (low-T supercond)
 - Heating systems
 - Remote Handling
 - Vacuum and pumping
 - Balance of Plant

7. Low cost

Low capital cost and long term technologies

8. Stellarator

2010

2020

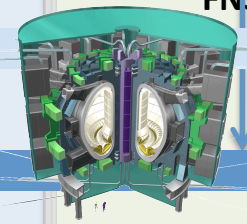
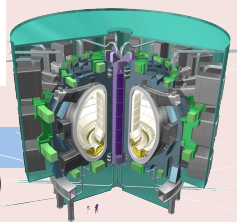
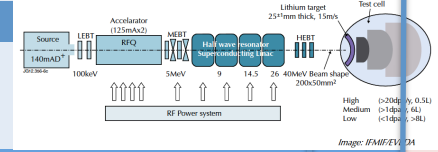
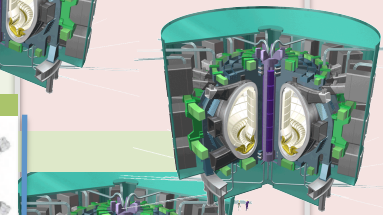
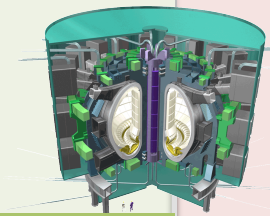
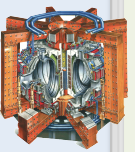
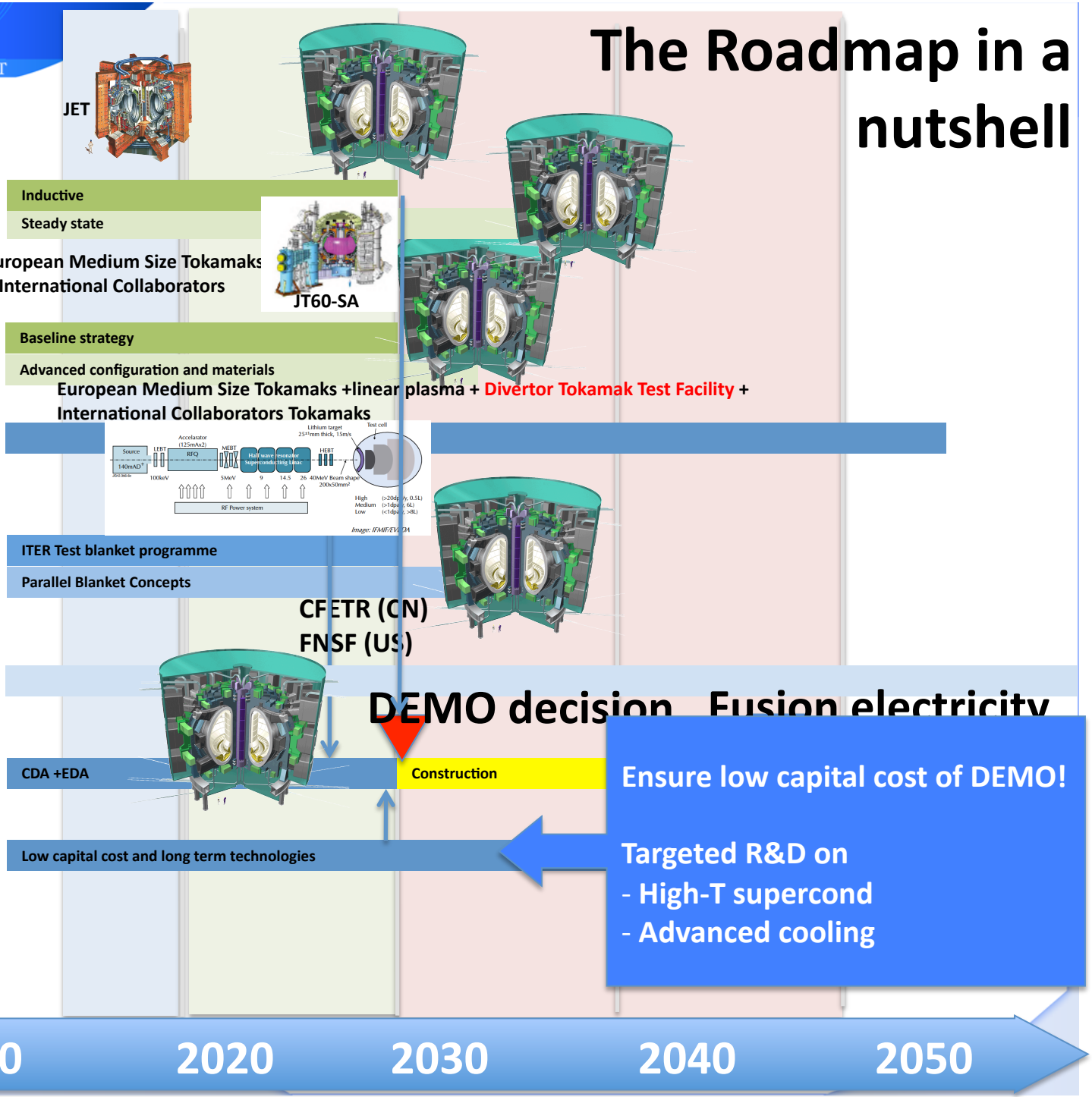
2030

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The Roadmap in a nutshell

1. Plasma operation

Inductive Steady state

European Medium Size Tokamaks + International Collaborators

JT60-SA

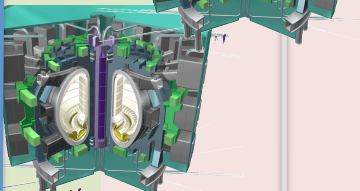


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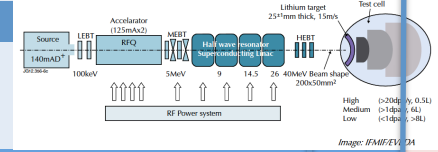
Baseline strategy

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3. Materials



4. Tritium breeding

ITER Test blanket programme

Parallel Blanket Concepts

CFETR (CN)

FNSF (US)



5. Safety

DEMO decision Fusion electricity

6. DEMO

CDA +EDA

Construction

Operation



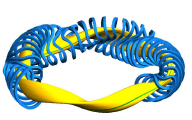
7. Low cost

Low capital cost and long term technologies

8. Stellarator

Stellarator optimization

Burning Plasma Stellarator




Conclusions

- The roadmap will be a living document, reviewed regularly in response to the physics, technology and budgetary developments.