

# International role of nuclear fission energy generation - status and perspectives

R. Stieglitz, J. U. Knebel, W. Tromm

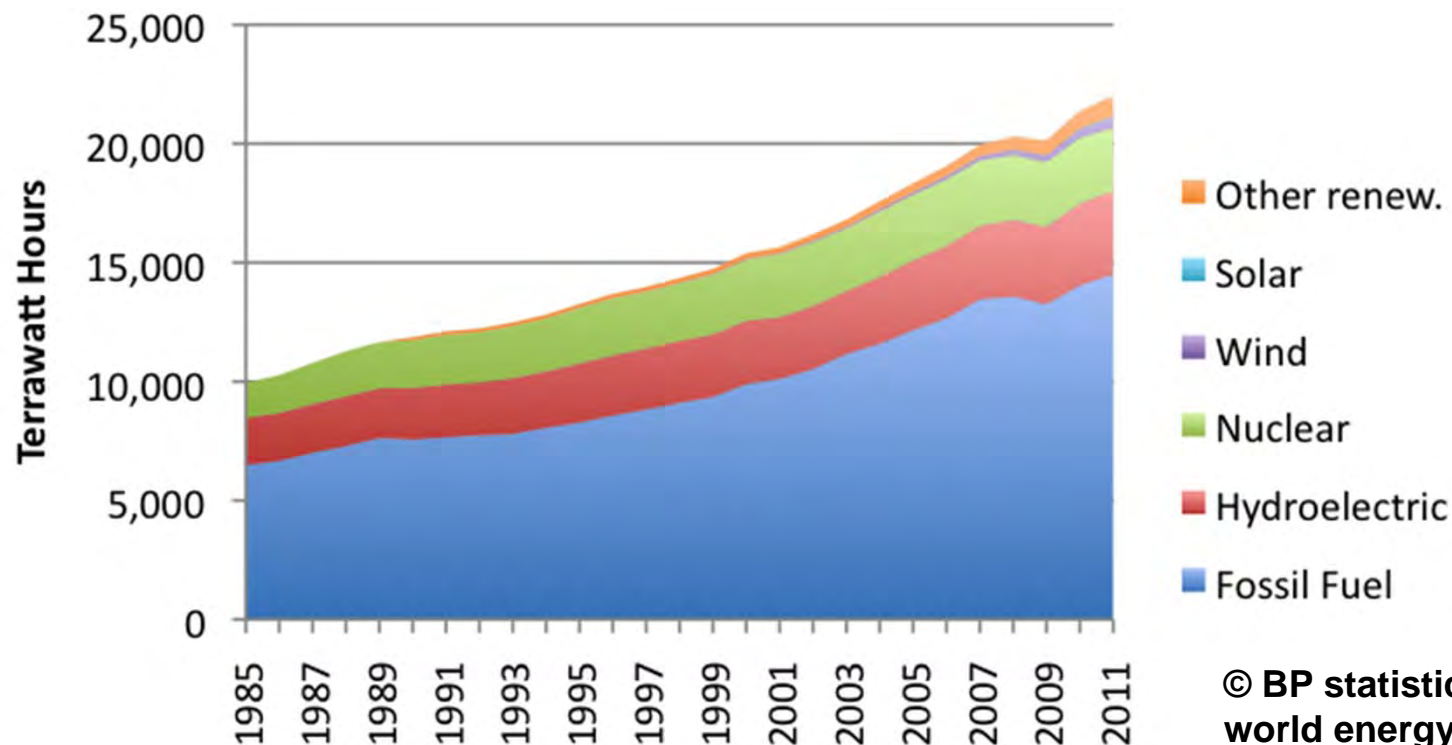


# Content

- **Present status of nuclear electricity generation – observations worldwide and in Europe**
- **Boundary conditions for NPP deployment-Large reactors (LR)/ vs. small medium sized reactors (SMR)**
  - **Economic considerations**
- **Safety concept of a NPP**
  - **General safety approach**
  - **Design safety**
  - **Severe accident safety & measures**
  - **LR under development**
  - **SMR technologies**
- **Generation –IV -Transmutation**
- **Some concluding remarks**

## Present status –Some facts

- NPP worldwide currently operating (3/2014, [www.iaea.org/pirs/](http://www.iaea.org/pirs/)):
  - 435 nuclear power plants commercially operated
  - 372 GWe net capacity
  - 72 reactors under construction
  - 240 research reactors in (56 countries), 180 nuclear powered civil ships
- Net electricity production 2370 TWh (2013)
- ➔  $\approx 11\%$  of global electricity production (almost constant since 2006)

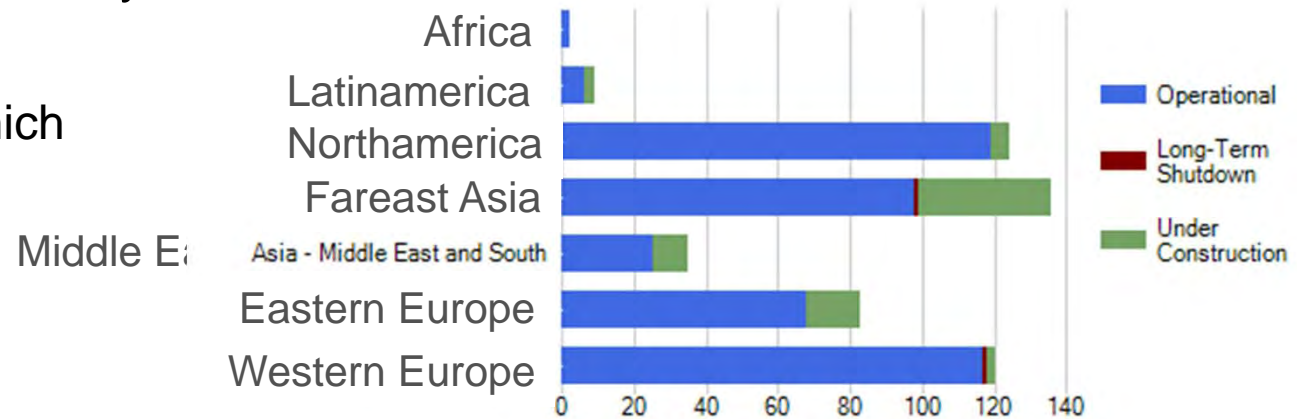


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# Present status –Some facts

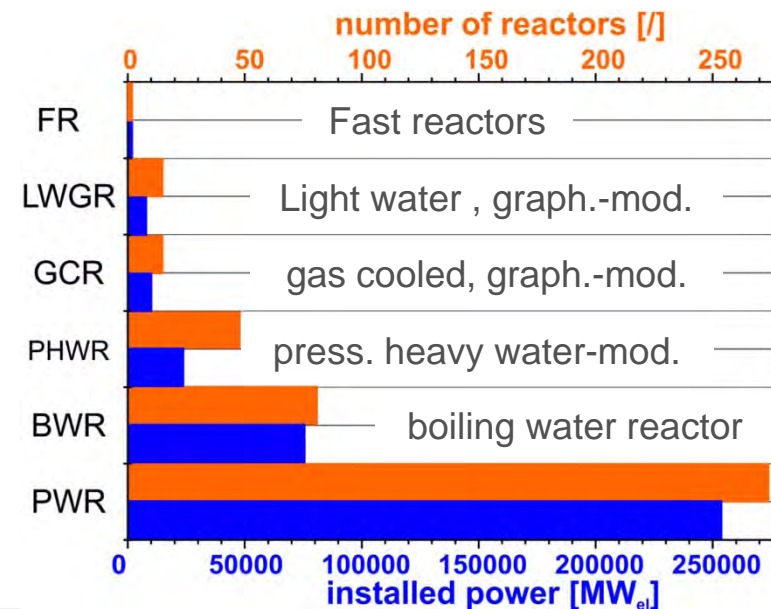
## Plant Location-currently-new builds

- ➔ 72 new builds of which
- ➔ 60 PWR's
- ➔ 68 GW<sub>el</sub>



## Reactor types-installed power

- ➔ focus on large scale units ~1GWe
- light water reactor (LWR)-types
- ➔ mainly pressurized water reactors (PWR)



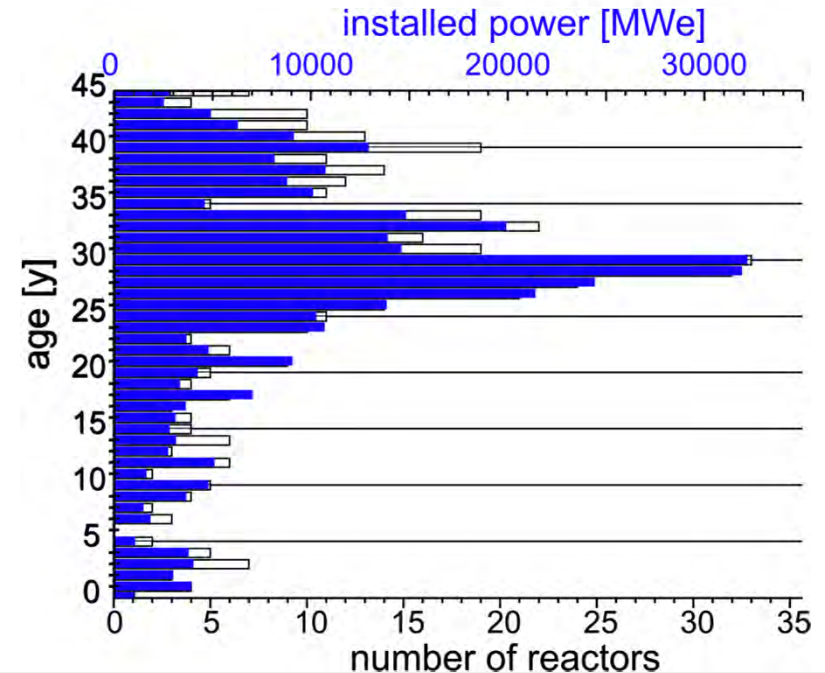
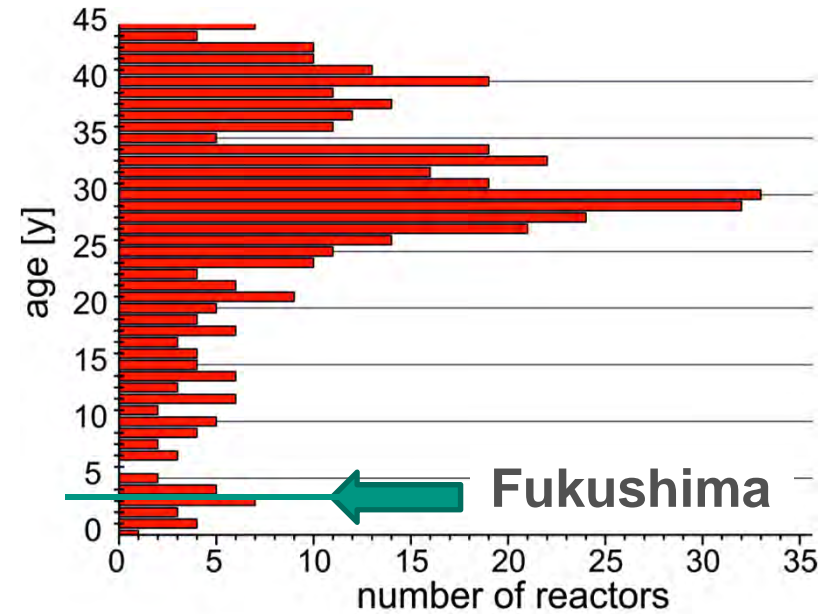
# Present status –Some facts

- Age distribution

- Mean reactor age ~30y
- ➔ Most reactors belong to Gen-II systems

- Nearly all current reactors operating are of LR-type

- Installed mean power >1GWe
- ➔ NPP operated as grid base load backbone



# Present status- Germany



## After march 11<sup>th</sup> 2011 Fukushima

- 9 NPP operating (12,068GW<sub>el</sub>)
- 8 shut-down
- 16 in decommissioning phase

## NPP electricity facts

- 97TWh<sub>el</sub> produced
- ➔ load factor (LF=) 92%
- Share in energy mix ~16%
- Difficult boundary conditions
  - Priority access of renewable energy sources (RES)
  - nuclear fuel tax
  - Regulatory constraints („stress test“,licensing, ....)

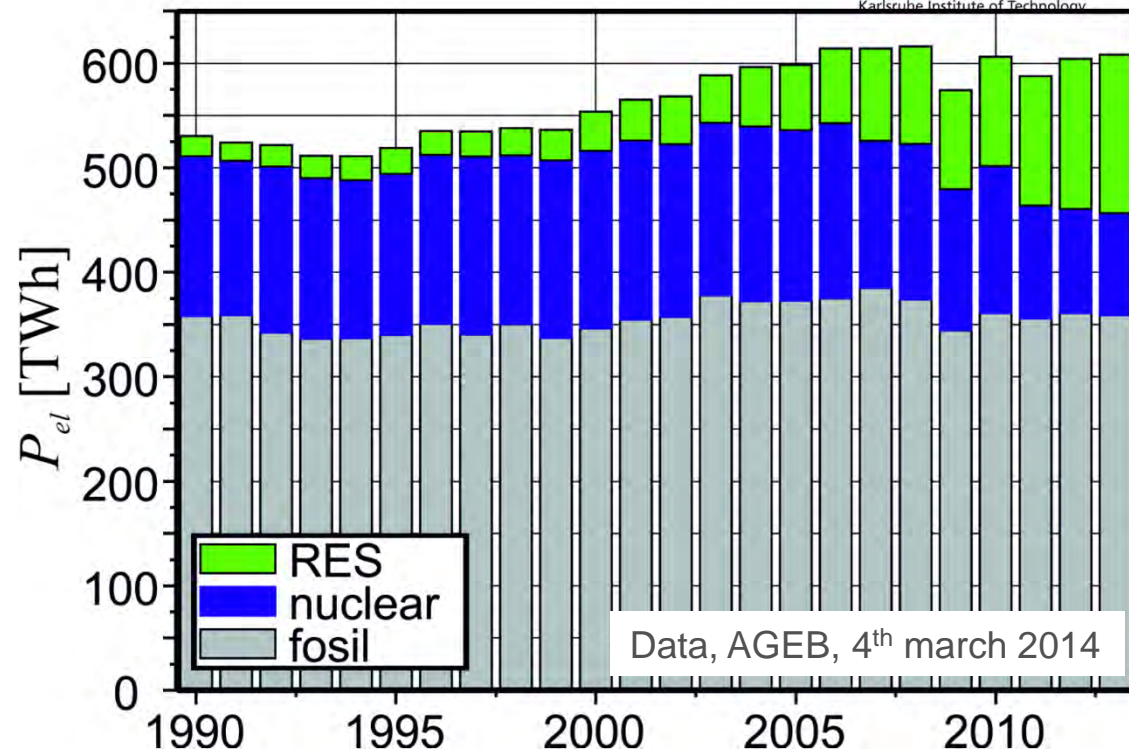


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# Present status- Germany

## Current German electricity share

- RES share 24.9%
- Installed capacity RES
  - 35,9GW Photovoltaics (PV)
  - 33,8GW Windpower
- Delivered RES energy
  - 30TWh PV (LF=9.5%)
  - 53TWh Wind (LF=18%)



## ➔ Successful „Energiewende“ demands

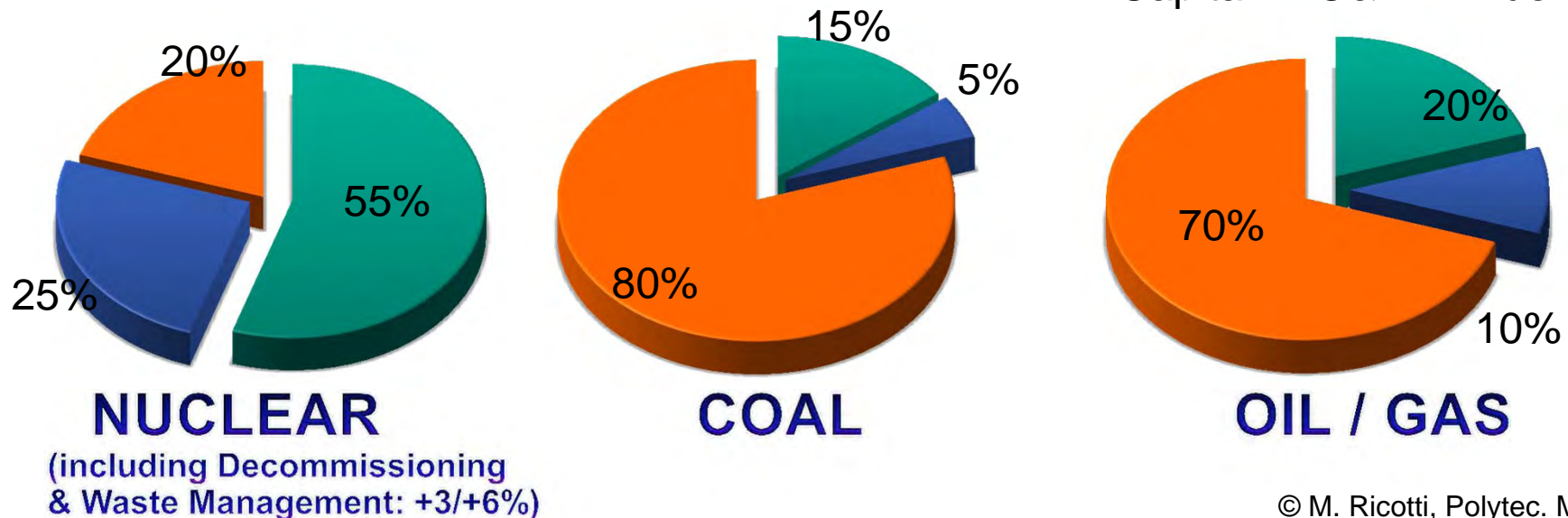
- transformation of grid **AND**
- provision of mature, reliable storage technologies

# Boundary conditions for NPP deployment

- **NPP deployment strongly dependent on national arguments**
  - Grid /electricity independence → autarchy (resources, availability,...)
  - Strategy of economic and social development → industrialization goals
  - technological basis → acceptance, perception
  - → maturity, safety performance, infrastructures
- Additional considerations: bridging technology ↔ long term option

## General facts

- Cost share of electric power plants





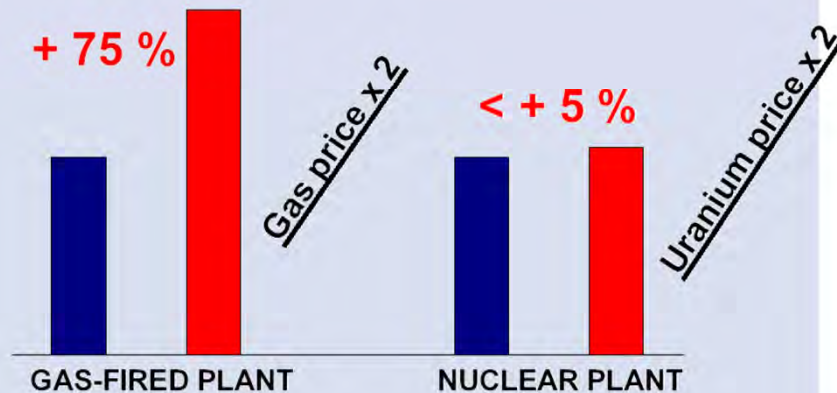
# Boundary conditions for NPP deployment

- Positive and negative effects in NPP erection

## POSITIVE



### Electricity Cost Sensitivity to Fuel Price Volatility



ThD / 7 May 2007

NUCLEAR POWER: GLOBAL STATUS AND PROSPECTS

16

## NEGATIVE

- Sensitivity to the Cost of Money
- construction delays/regulatory burdens
- capital intensive investment = exposure to market risk



# Boundary conditions for NPP deployment

- High capital investments
- Long construction schedule

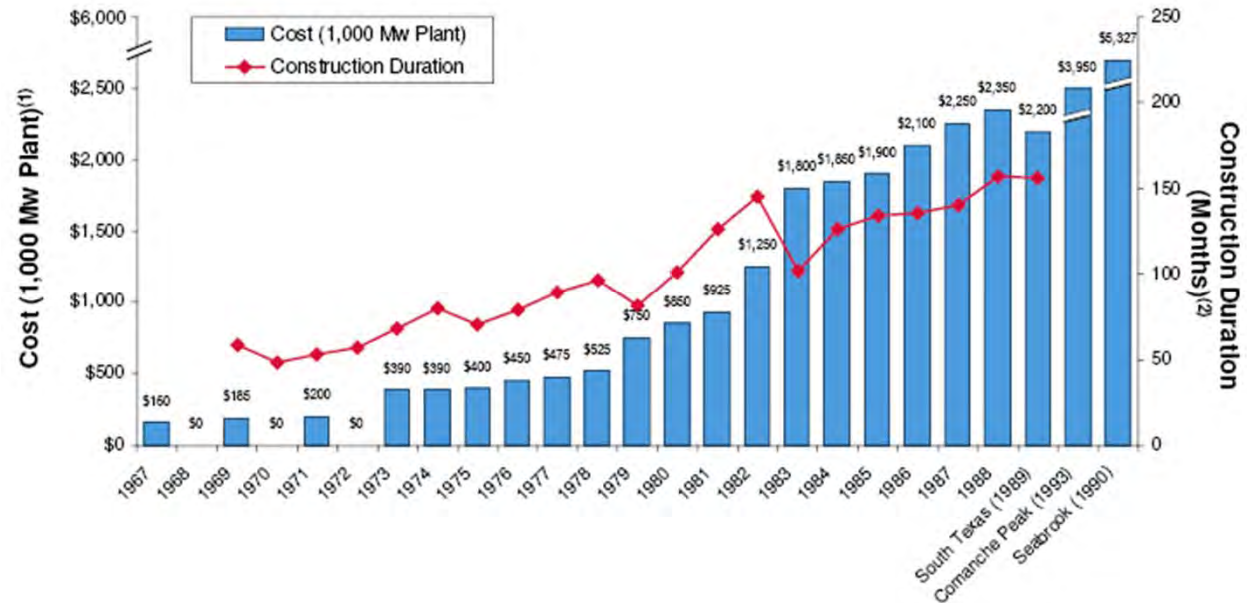


- High financial exposure
- Long Pay Back Time
- High investment risk



## Consequences

- Long-term investment strategy
  - stable energy politics environment
  - societal economic stability AND acceptance
- ➔ Especially for private operators in liberalized markets based on competition



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# Boundary conditions for NPP deployment

- Large reactors or Small Modular Reactors (SMR) ?

## Arguments for SMR

- flexible power generation → wider user/application range
- replacement of fossil fired units
- enhanced safety margin by inherent and/or passive safety features;
- better affordability - freedom in upgrading
- Cogeneration & non electric applications (desalination-process heat),
- Hybrid energy systems composed of nuclear with RES.

**But deployment & technology of SMR is not**



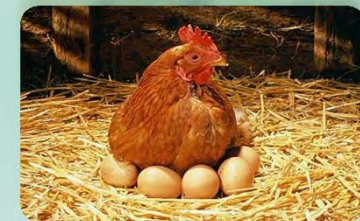
simply a scale reduction



=



sum of the modules



**= different product & technology**

# Boundary conditions for NPP deployment

## LLEVELIZED UUNIT EELECTRICITY CCOST = LLUEC

- Calculated as “Lifetime levelized cost”
- Sum of cost items:
  - Investment cost including capital remuneration
  - Fuel cycle (front-end and back-end)
  - Operation & Maintenance (O&M)
  - Decontamination and Decommissioning (D&D)

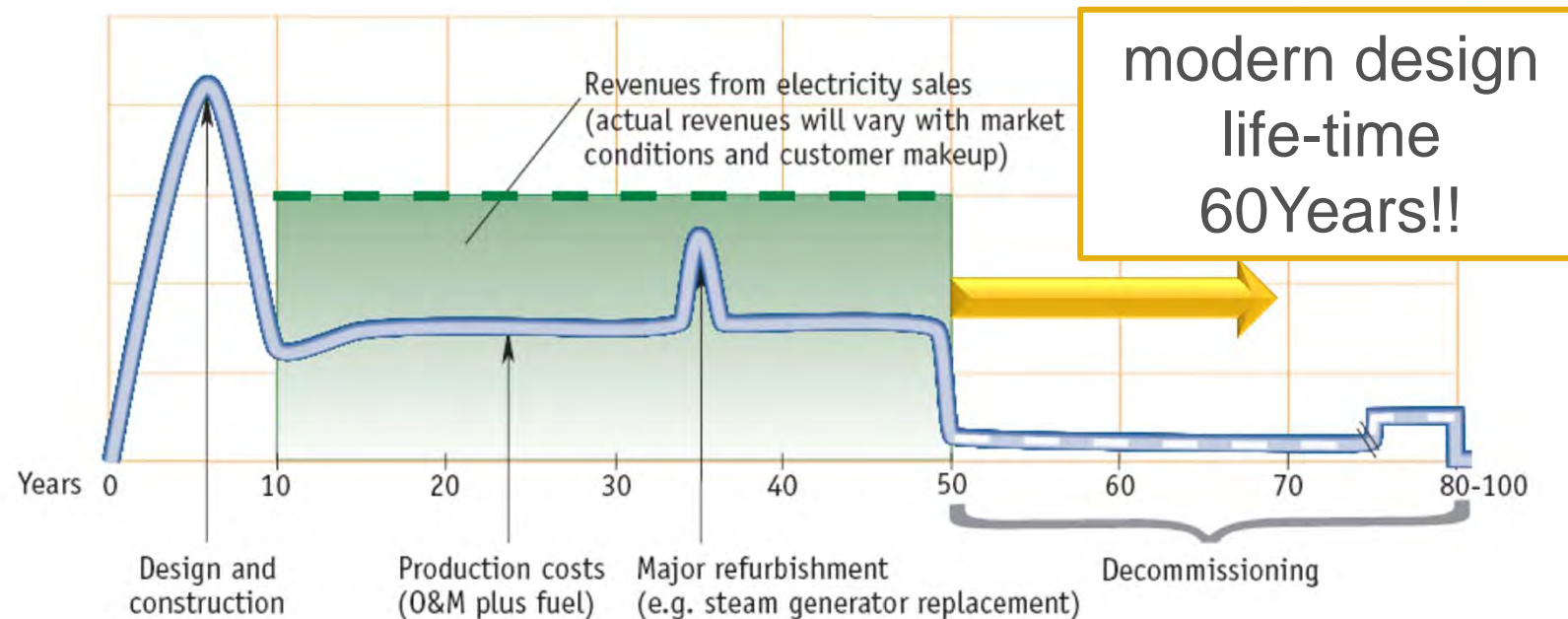
€/MWh

INVESTMENT

FUEL

O&M




D&D

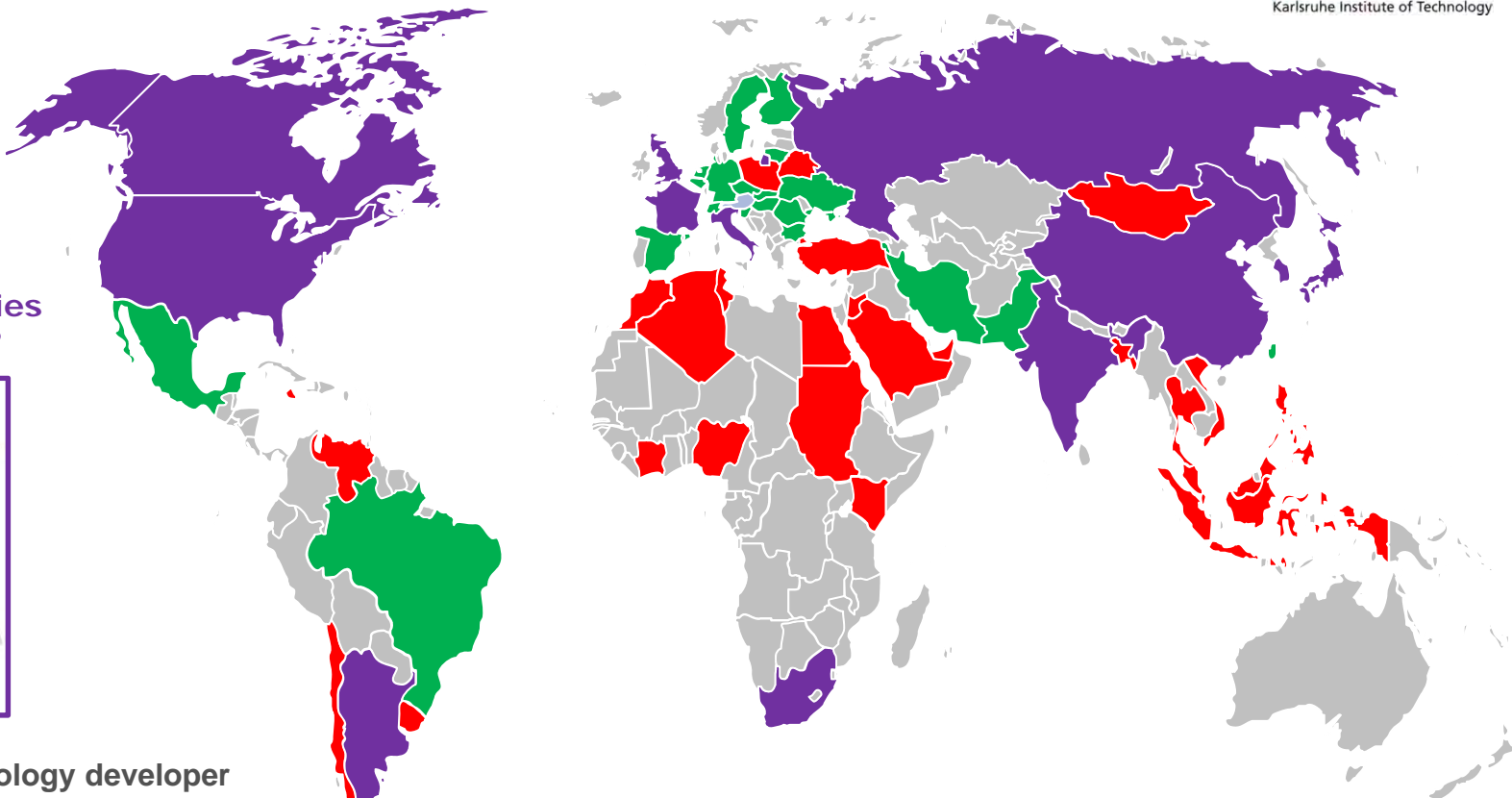


# Status of Countries on Nuclear Energy Initiatives

Which countries deploy SMRs?



-  Technology developer countries (with NPPs in operation)
-  Other countries with NPPs
-  Newcomer countries



© Subki, IAEA, 2012

# Major aspects for nuclear reactor deployment

## ■ Currently deployment of Gen III –reactors

Are they essentially new compared to running Gen-II types? **-No**

➔ Evolutions of the operating Gen 2 plants

## Why ?

### ■ Low industrial risk:

- Include feedback of experience of the global fleet
- Designed on well proven physics principles
- No technological leap necessary

### ■ Performance vs. sustainability = Gen 2

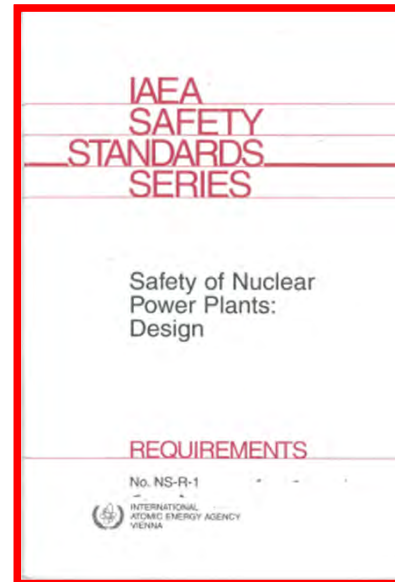
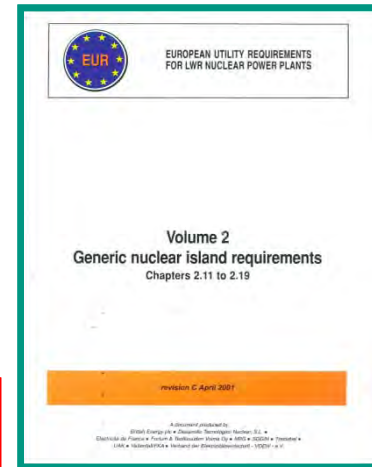
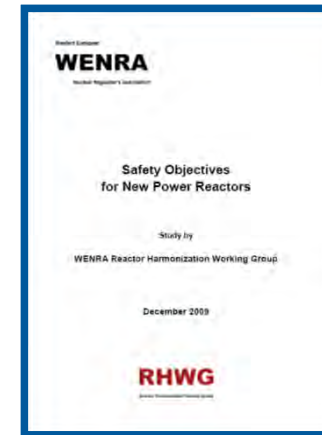
# Major aspects for nuclear reactor deployment

- Hardened **design objectives** for
  - **nuclear safety** (Severe accident integrated in design; limited radiological consequences, Core damage frequency  $<10^{-6}$  /y, more robust defence in depth approach -diversity, specific measures for each DiD level, integration of external events and hazards in safety concepts)  
and
  - **public acceptability** (No area submitted to off-plant emergency planning, Low environmental impact in normal operation and design basis  
**after Chernobyl (1986), NewYork (2001) and Fukushima**)
- Hardened **economic design objectives** (competition with other sources)
  - **profitability of project** (availability  $>90\%$  along life-time, short refuelling- outages, long cycles, reduced investment → large size, design simplification, construction duration)
  - **Investment protection** (lifetime 60-80 years, low rate of difficult-to-repair failures, low core melt frequency  $< 10^{-5}$ , proven technology → no leaps)
- **Gen-III reactors are not Gen 4 !!!**
  - No design requirement(s) for sustainability (saving  $U_{235}$  resources)
  - No burning of minor actinides

# Requirements quite well established & documented

Numerous standards posed in documents by

- utilities,
  - national TSO,
  - Regional within the EU and
  - worldwide collaborations
  - and through IAEA
- and continuously updated.





# Safety concepts of NPP's-General

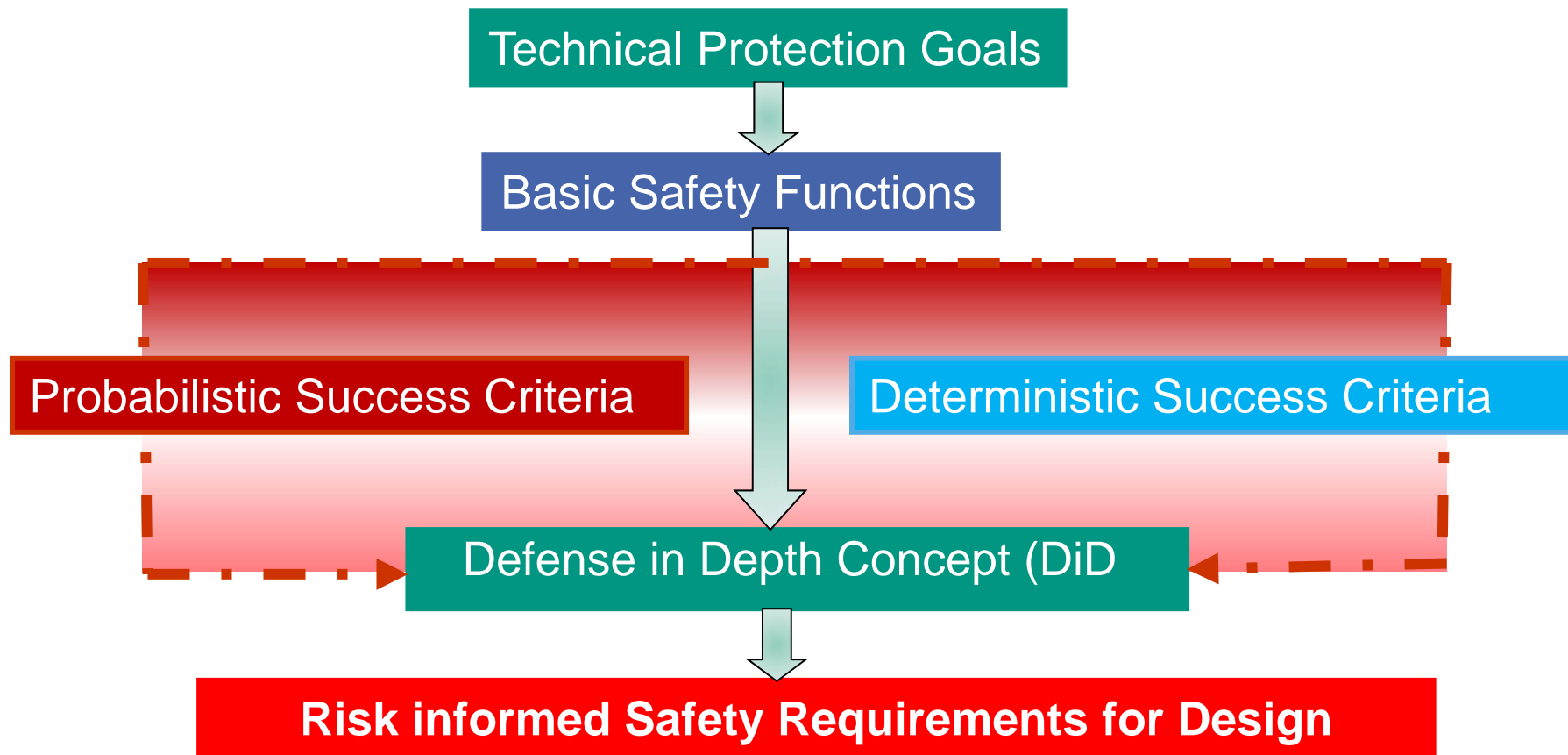
## Major protection goals for NPP to be matched by design

- Confinement of radionuclide inventory
- Coolability at any time irrespective of origin and source
- Control of reactivity
- ➔ Defence in Depth (DiD) approach ➔ assignment of safety levels

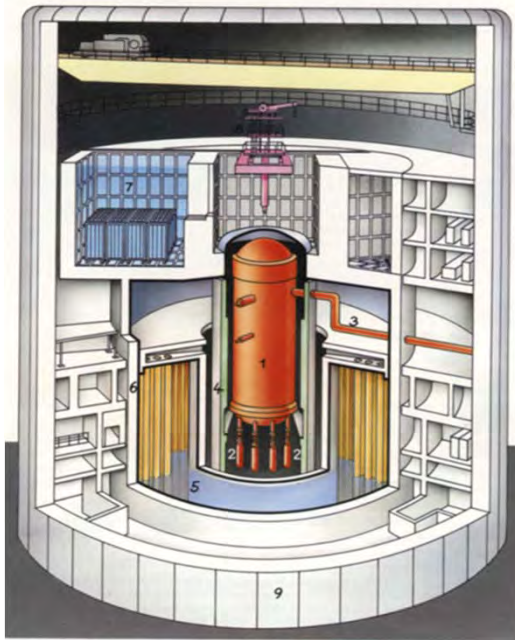


lev.	cond.	aim	measures	consequences
1	normal	prevention of anormal operation or failures	Conservative design, high quality construction, qualified personnel	No measures
2	operational failure	condition control, detection/identification of reason	Control, limitation/ protection measures and survey functions	After short time restart
3	Design basis accident (DBA)	control of DBA within design (e.g. multiple failures of safety functions)	Engineering safety charact. and implementation of controlled accident measures	Planned restart anticipated (after inspection, repair, qualification)
4	Severe accident (BDDBA)	Control of critical plant states incl. prevention of propagation	Complementing measures and accident management	<b>Re-start not required</b>
5	Post severe accidents	Mitigation of radiolog. consequences	Off- plant emergency measures	<b>No plant re-start assumed</b>

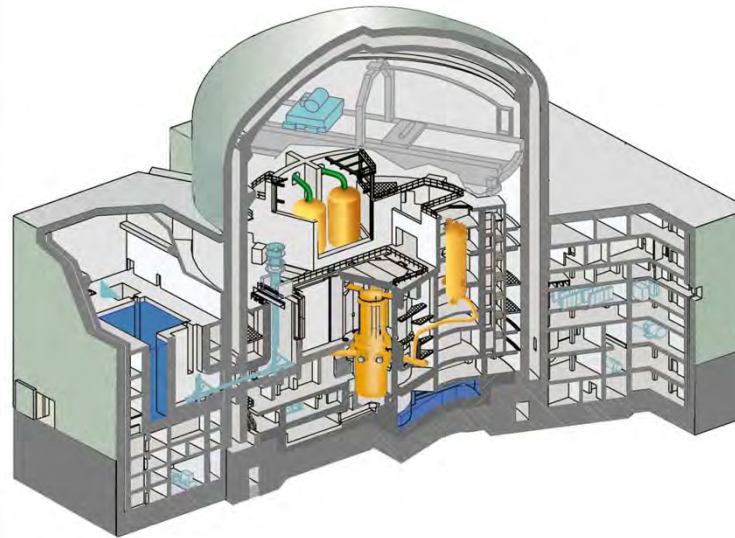
# Safety approach- Risk informed safety philosophy



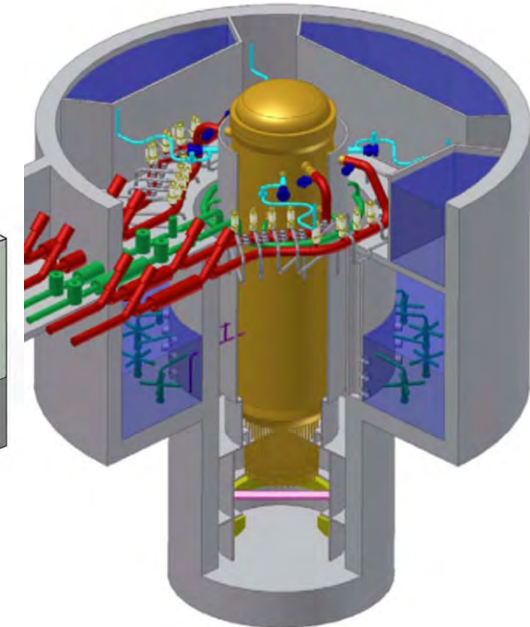
# Design basis safety: Gen II and Gen- III Reactors



■ BWR



■ EPR-PWR



■ ESBWR

## **NPP: Complex System with Multi-physic and Multi-scale Phenomena**

**Main challenges for risk informed safe design :**

- Neutronic, thermal hydraulic, mechanical design – **ALL ARE COUPLED**
- Passive safety systems for ECC and decay heat removal
- Control of severe accidents (core-catcher, passive containment cooling, PAR)

# Design basis – safety

## Enlarged computational capabilities and resources allow for

- ➔ more detailed local analyses in the reactor design
- ➔ improved design safety of new plants (Gen III )
- ➔ retrofitting of running plants (Gen II)

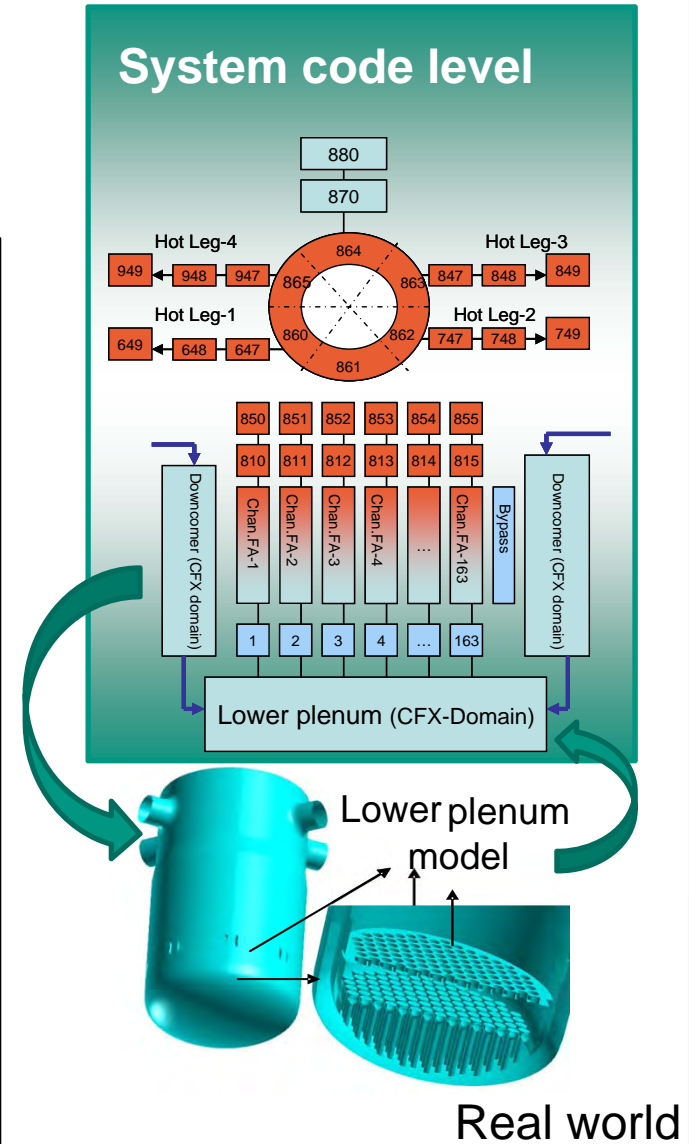
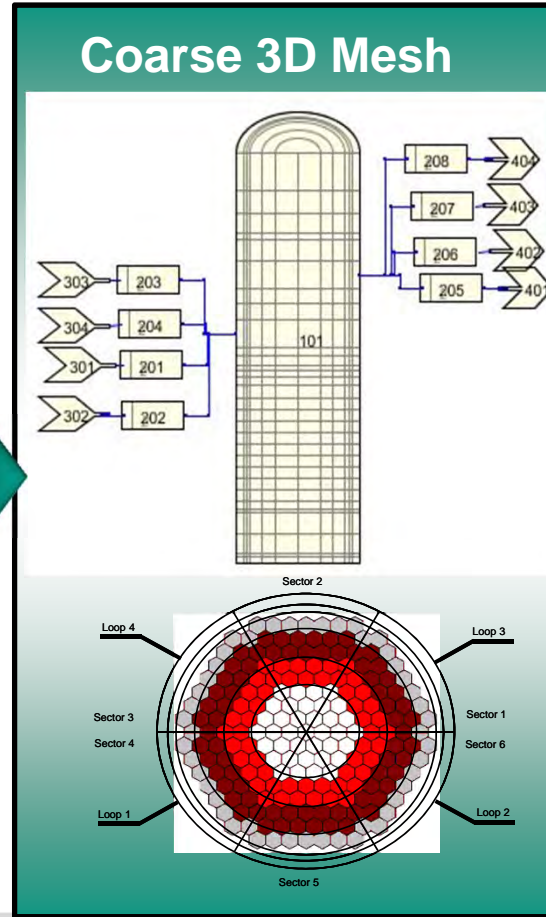
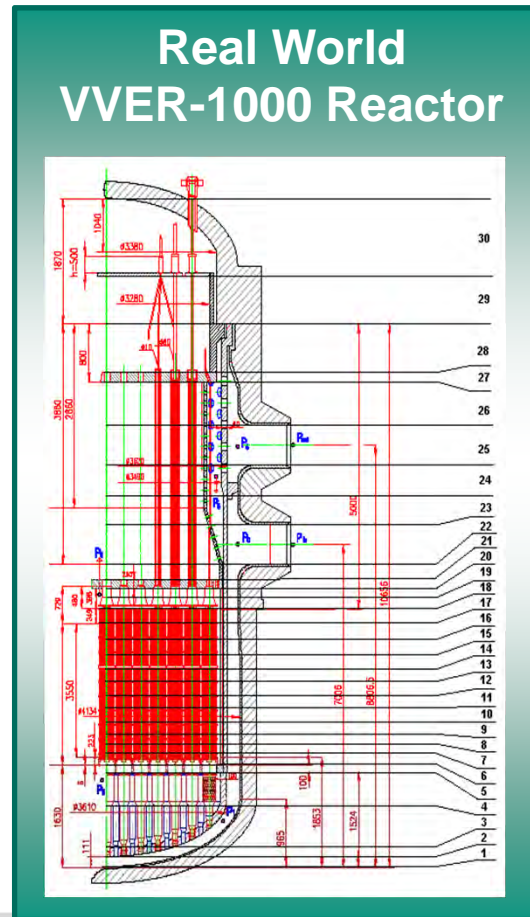
## Recipe to solve the sophisticated problem involve:

- Multi-scale problems
- Multi-physics problems
- Multi-scale and multi-physics
- including transients
- ➔ **A very challenging problem with numerous feedbacks !**

# Design basis – safety

## TH- problem – „classic route“

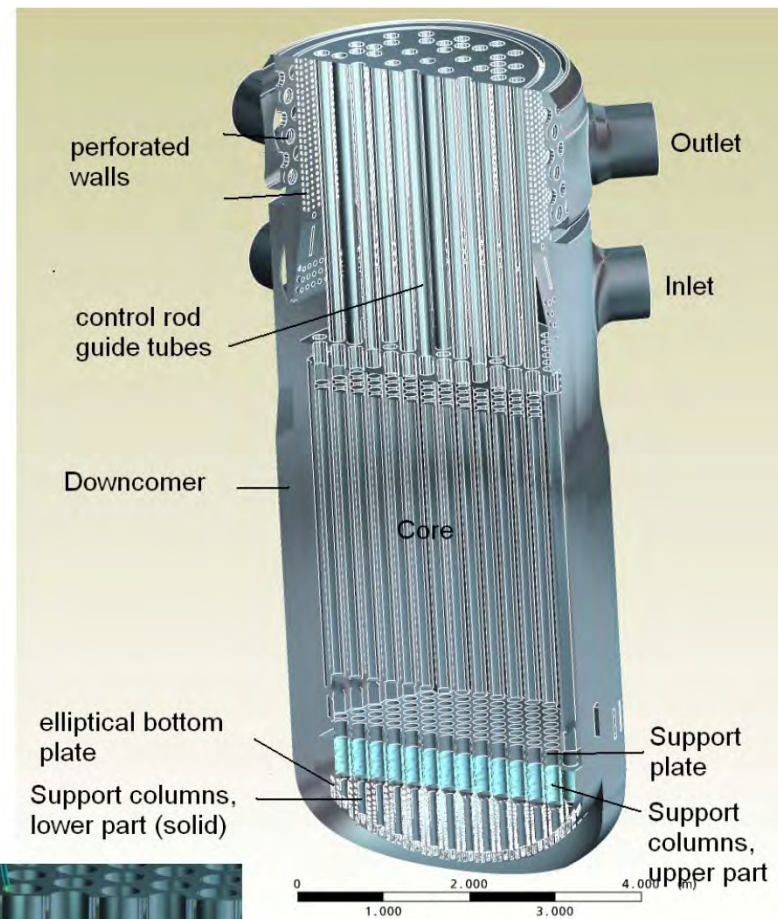
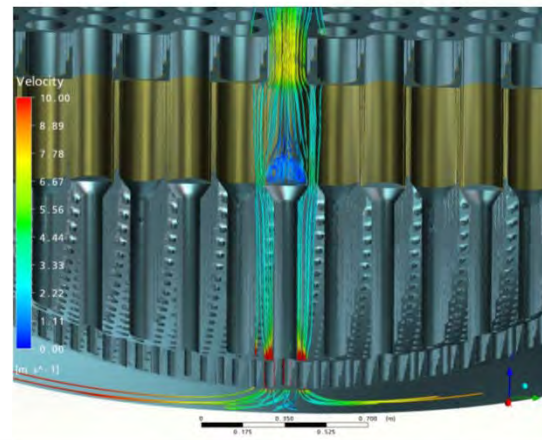
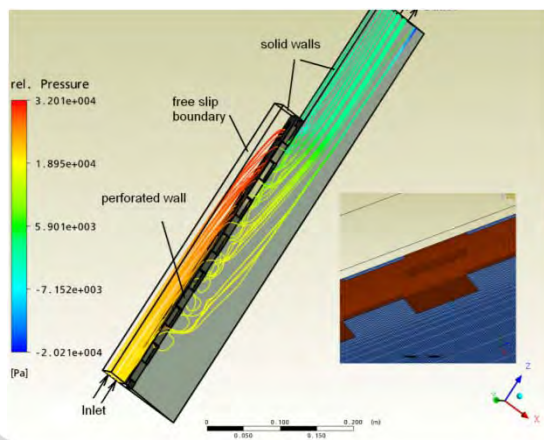
- Fast running real time capability
  - reactor operation
  - principle design



# Design basis -safety

## TH- multi-scale –problems –CFD Flow in reactor pressure vessel (RPV) micro → macro scale

- Down comer and lower plenum:
- Computing effort 2 weeks CPU time (12 processes parallel) for 1800s transient
- Development chain
  - $\Delta p$  obtained from standalone full detail model (3 Mio cells / column)
  - Implementation of  $\Delta p$  coefficient in the coarser RPV model (5000 cells / column)



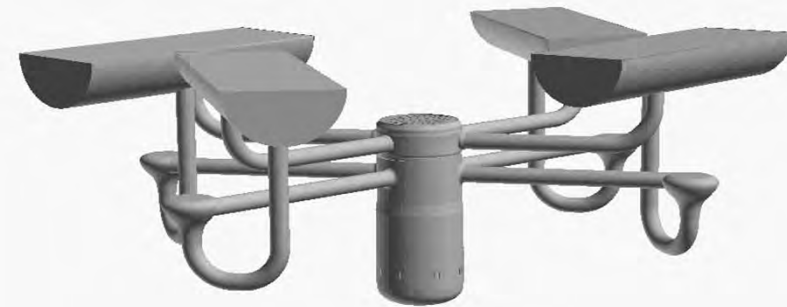
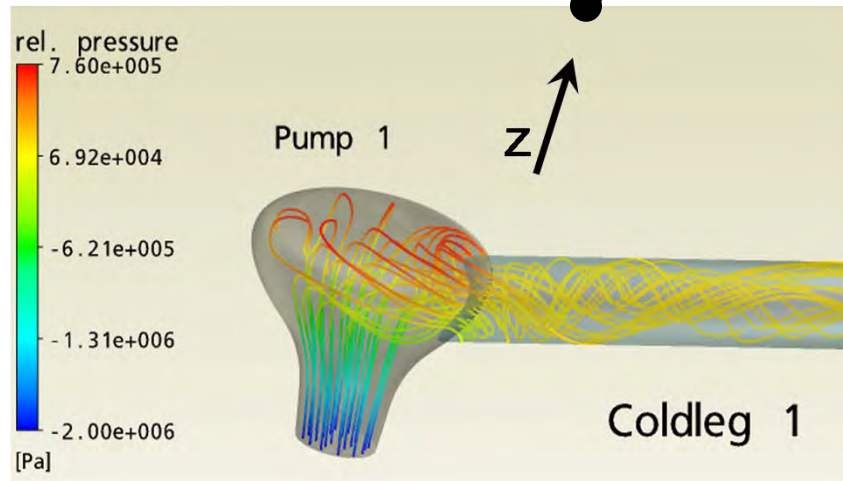
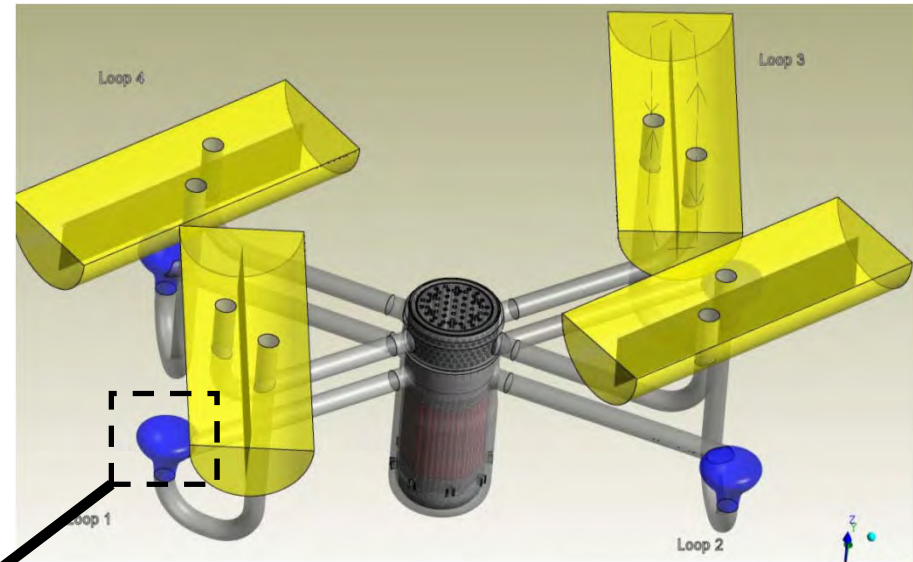
VVER-1100 reactor

# Design basis -safety

TH -multi-scale -problems

RPV → Primary loop (VVER-1000)

- RPV
- Heat exchanger
- Primary loops:
  - Steam generators and pumps
  - Pipes
  - Valves



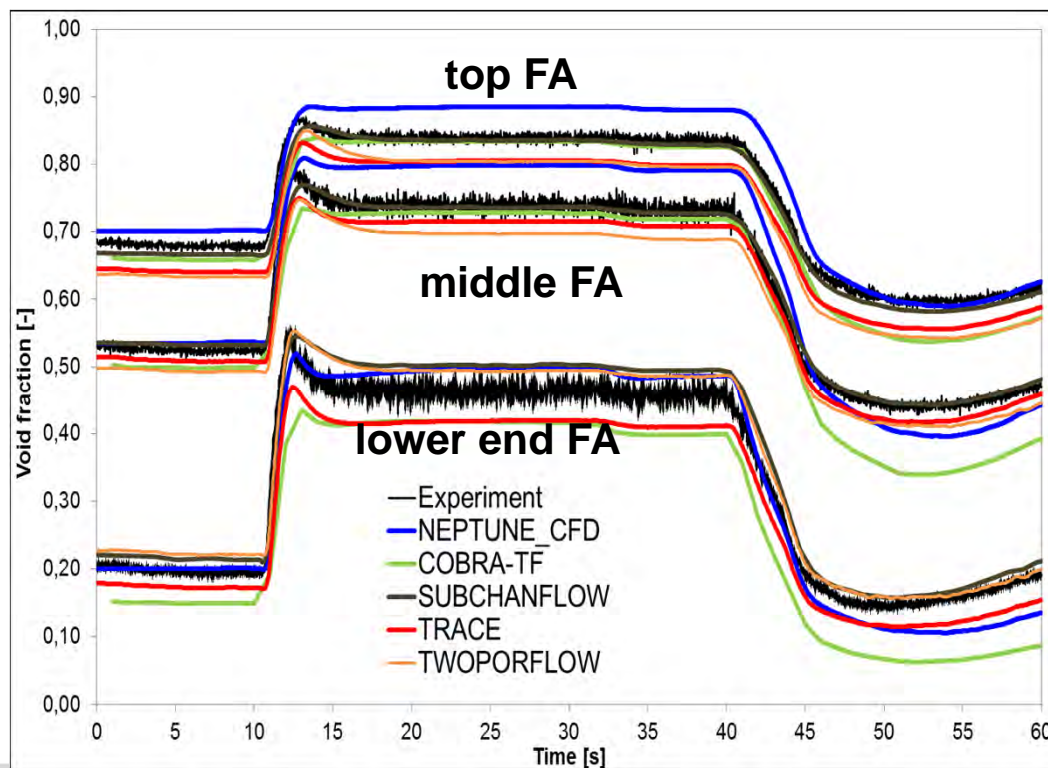
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# Design basis -safety

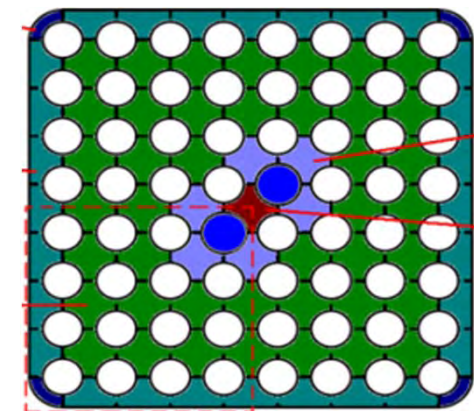
- TH Validation essential corner-stone → IAEA –Benchmarks

## Example:

- OECD/NEA Benchmark: Pump Trip exercise
  - Void fraction
  - Pressure drop
  - Critical power



## Fuel assembly (FA)



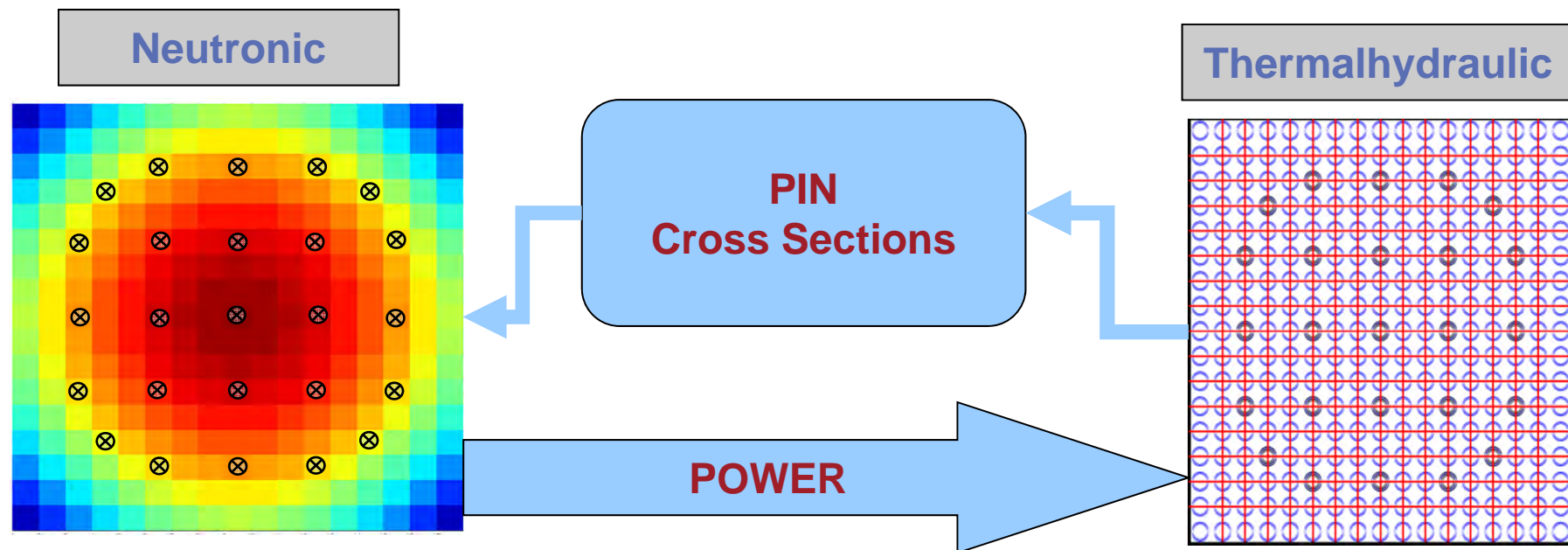
© Perez-Manes PhD Thesis 2013



# Design basis -safety

## Advanced methodologies for the analysis of PWR and BWR Transients

- Coupled thermal-hydraulics and neutronics
- High-fidelity / multi-physics developments: from FA to pin-based solutions
  - Direct prediction of local safety parameters at cell level
  - Reduction of conservatism

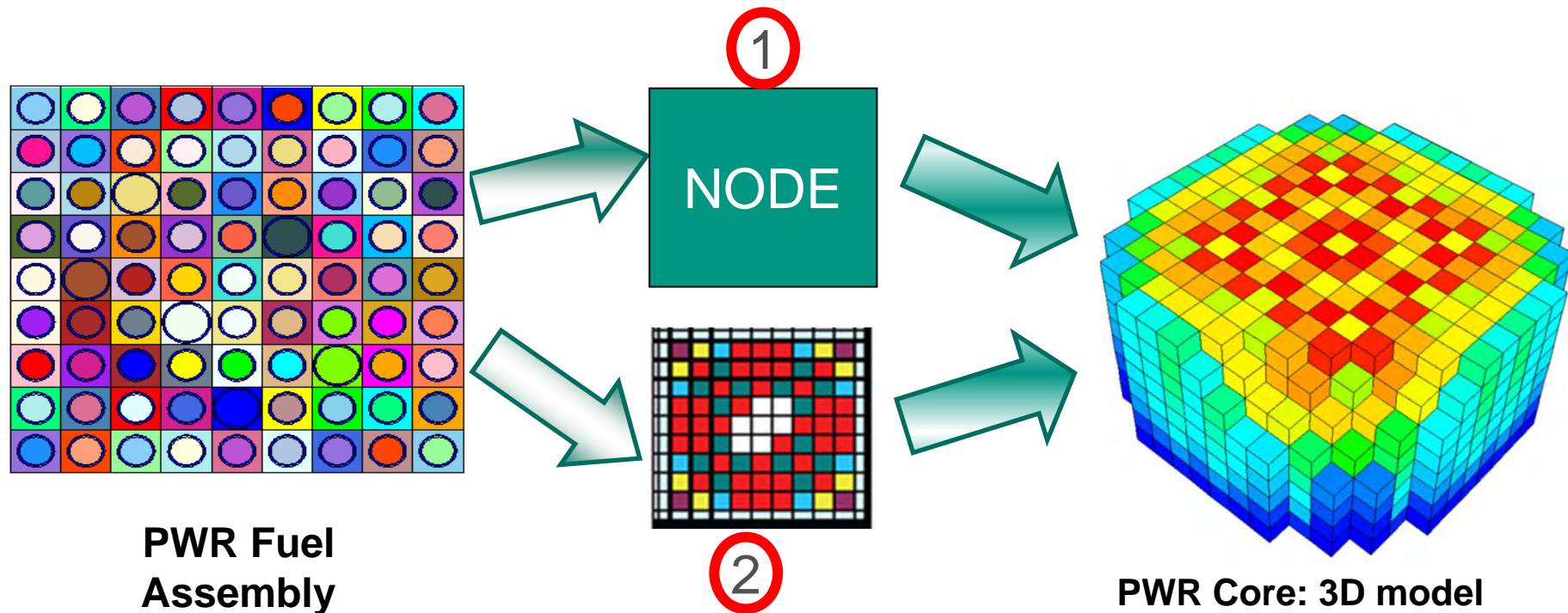


# Design basis -safety

## Actual Trend: Multiphysics and multiscale problems

### “Two routes”

- Fuel Assembly level simulations → conservative safety parameters **1**
- Pin level simulations → local safety parameters, but costly **2**
- economic AND save designs demand high spatial resolution on core level



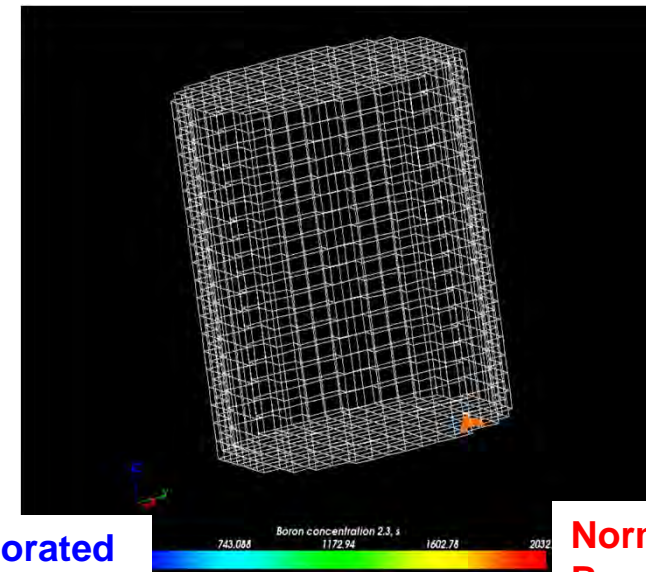
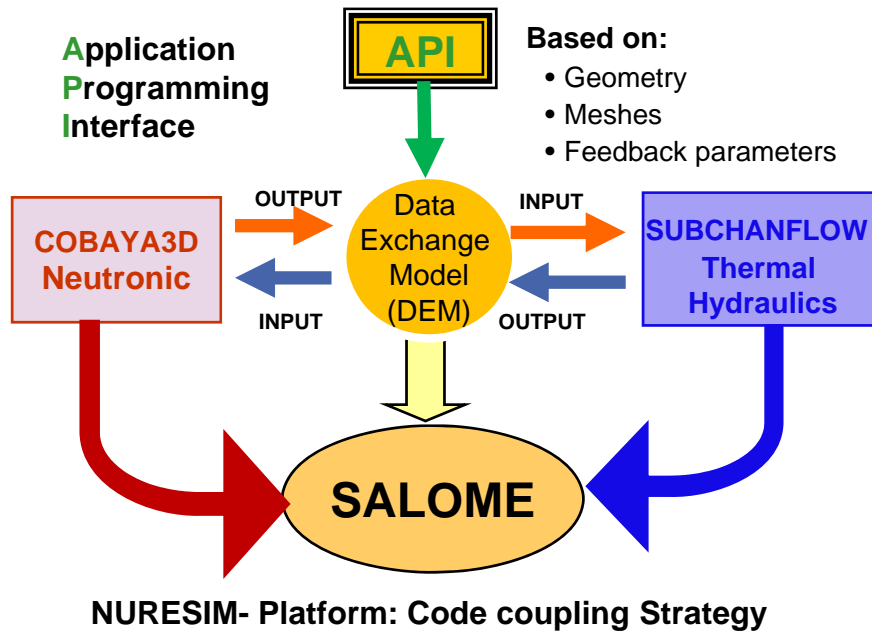
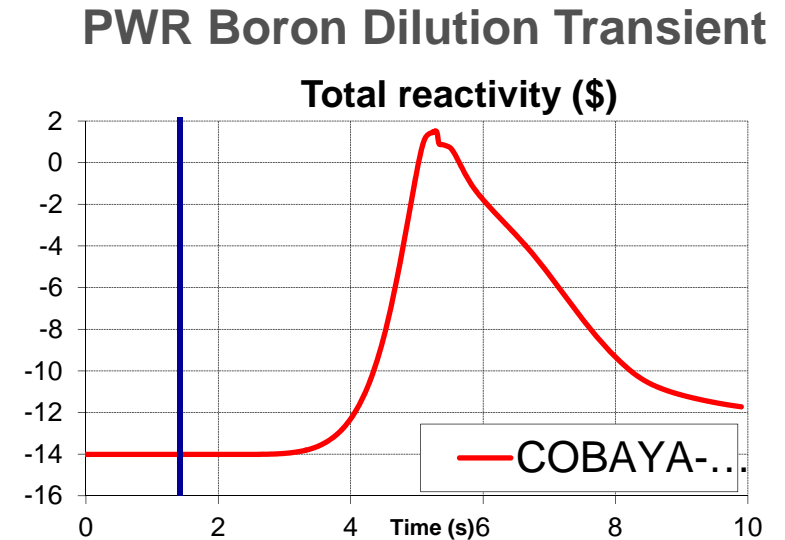
# Design basis -safety



## Actual Trend Multi-/scale -physics

1

- ➔ local FA or even pin data
- Mesh super-position at FA level with pin-power- reconstruction
- Demanding High Performance Computing (HPC) and parallelization



© Calleya PhD Thesis 2013

# Design basis -safety

Actual Trend:

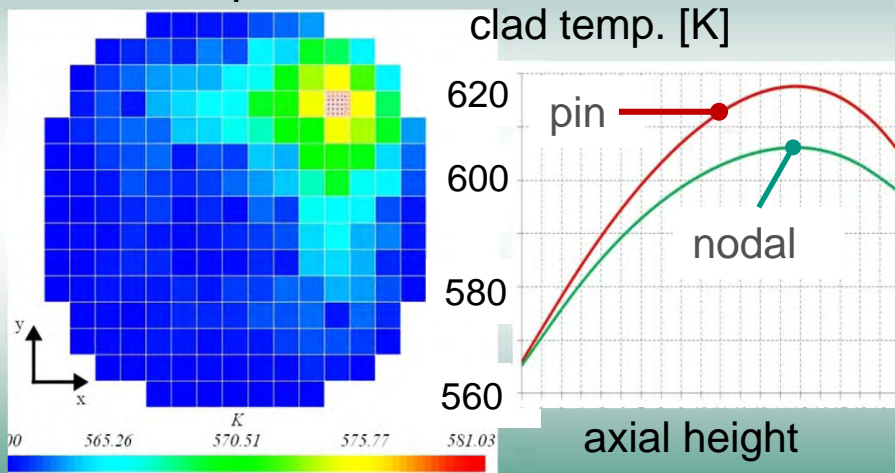
Multiphysics and multiscale problems

① - ②

②

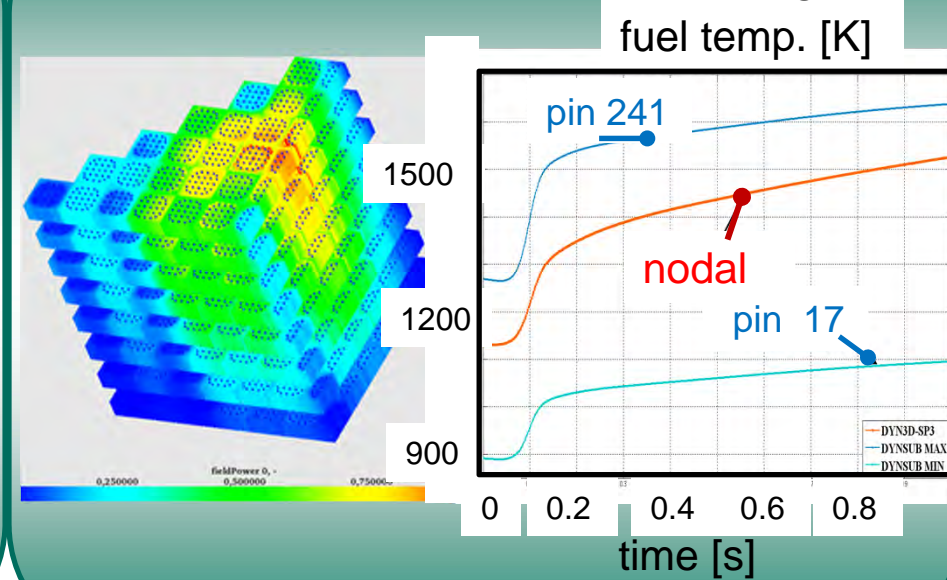
## Hybrid schemes

- Nodal in most of core
- Local pin resolution



## Pin resolution

→ computational demanding



© Ivanov, PhD Thesis 2014

Next steps underway → tracking each neutron → Monte Carlo methods

# Beyond design basis -safety

- Integral part of Gen-III reactor design



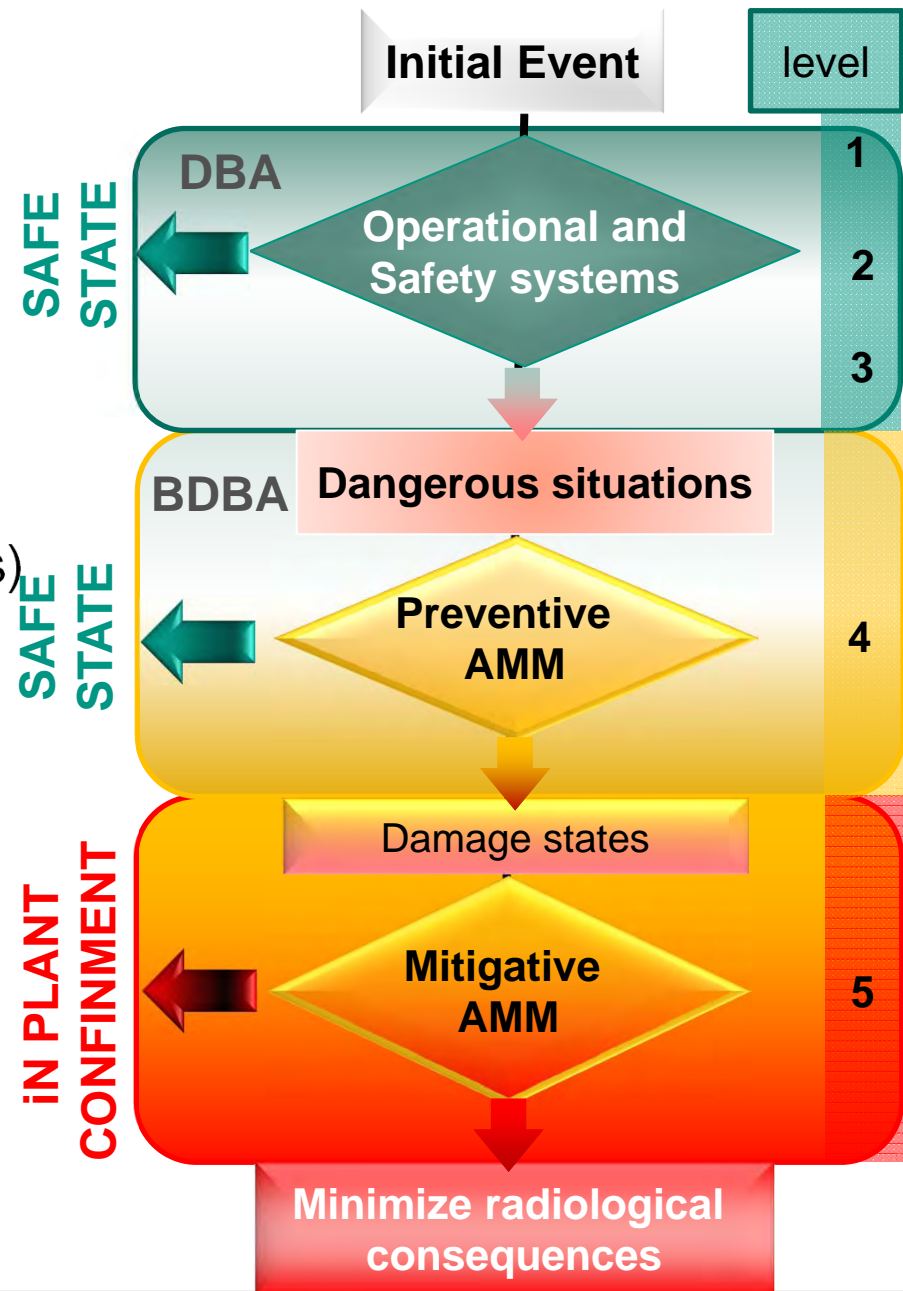
## What to be avoided ?

- Fukushima (→ radiolog. consequences)



## Design options

- Design, core catcher, PAR,
- Barriers,.....



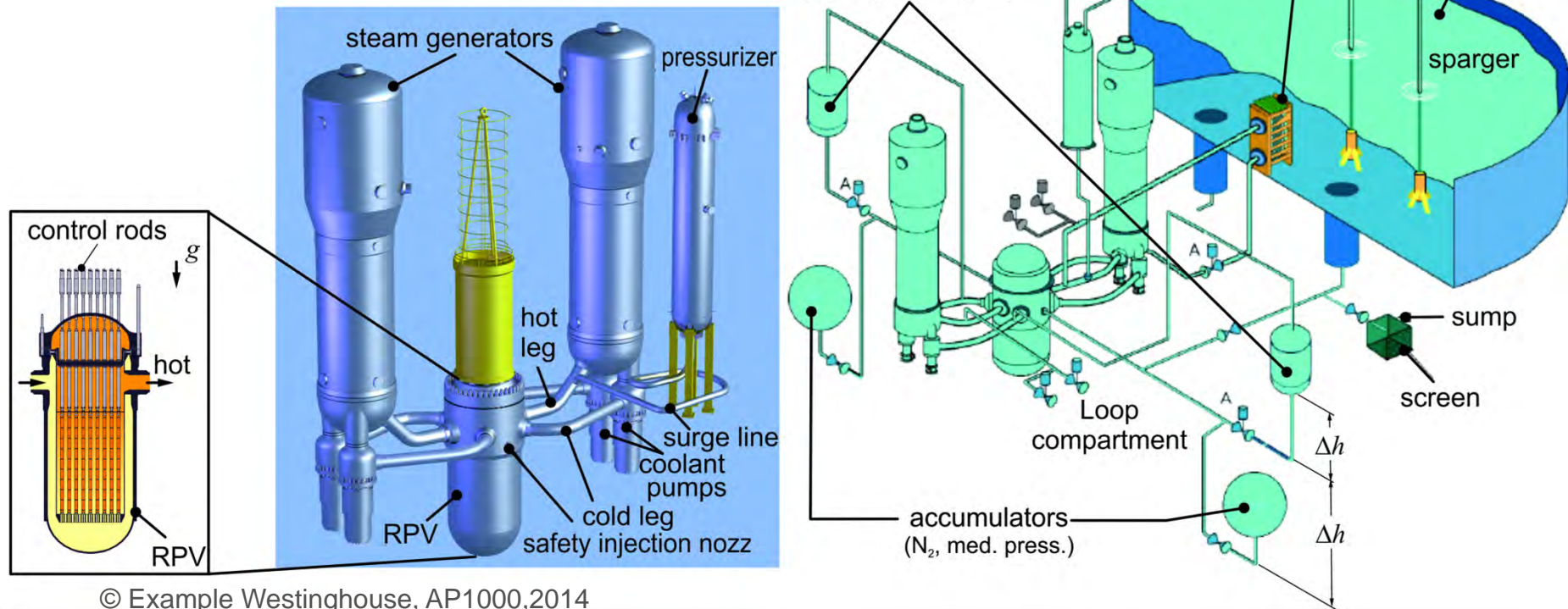
# Standard NPP Safety Systems- Gen II

## Control

- Control rods
- Borated water

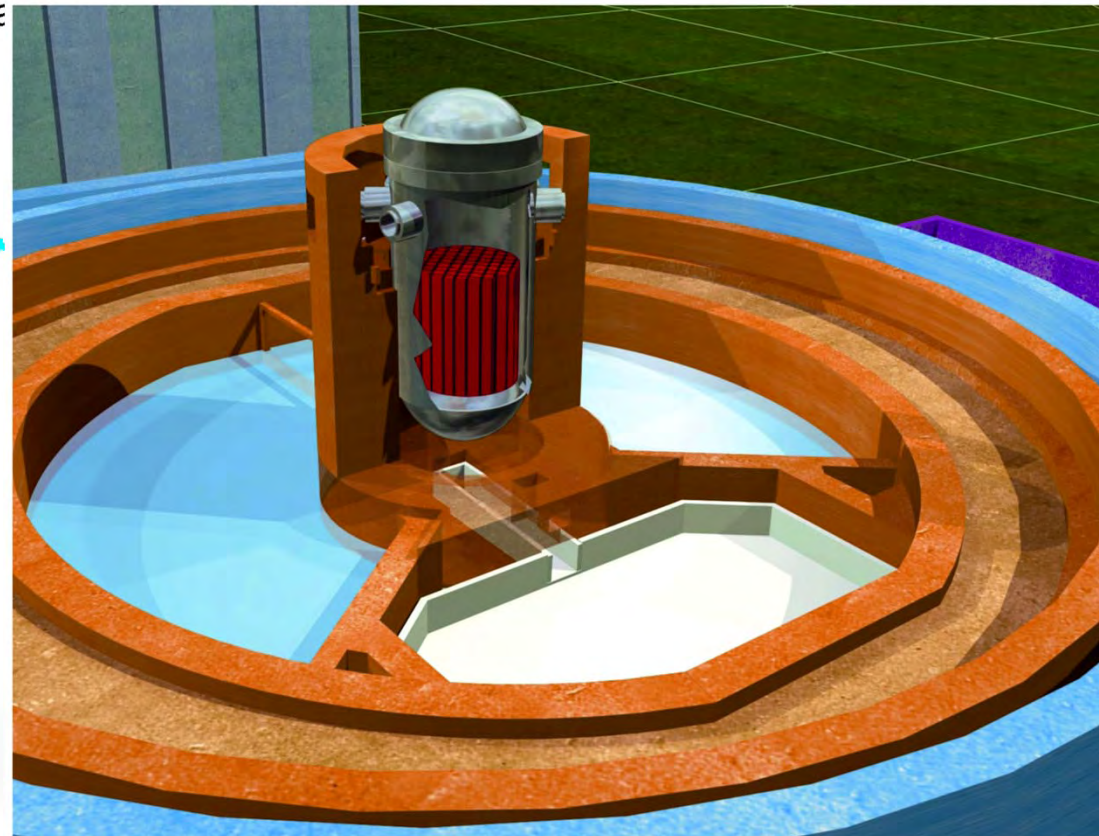
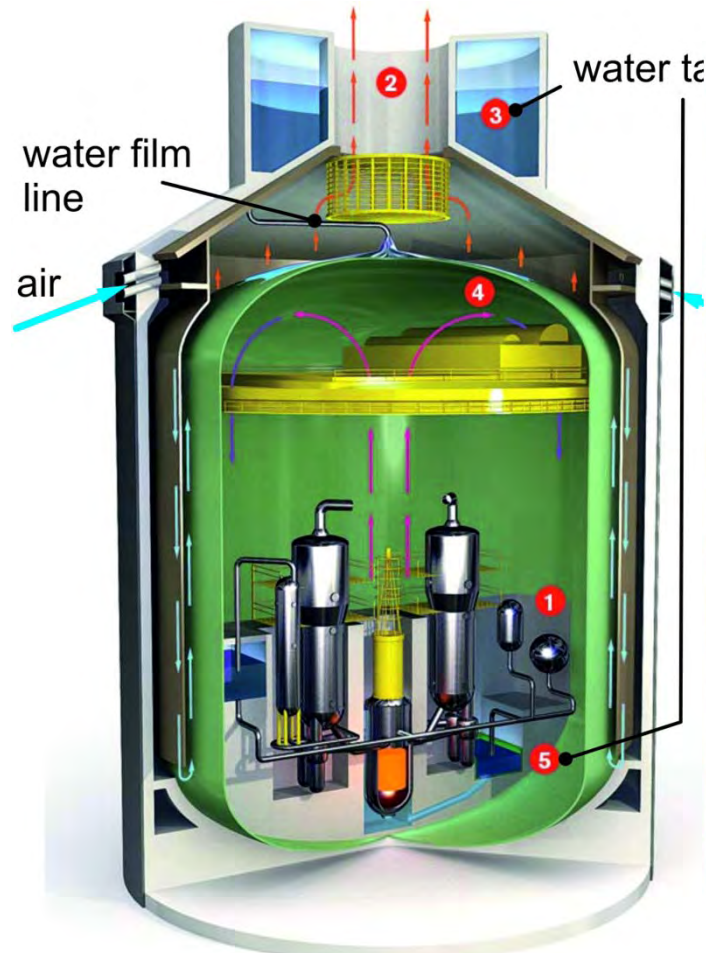
## Purely passive and safety related Emergency core cooling systems (ECCS)

- Core make-up-tanks (borated water)
- Accumulators (water replacement)
- Coolant make-up from IRWST by gravity
- PRHR gravity based



# Evolutionary Safety Systems- Gen III

- Several severe accident strategies
- In-vessel retention
- ex-vessel by means of „core catcher“



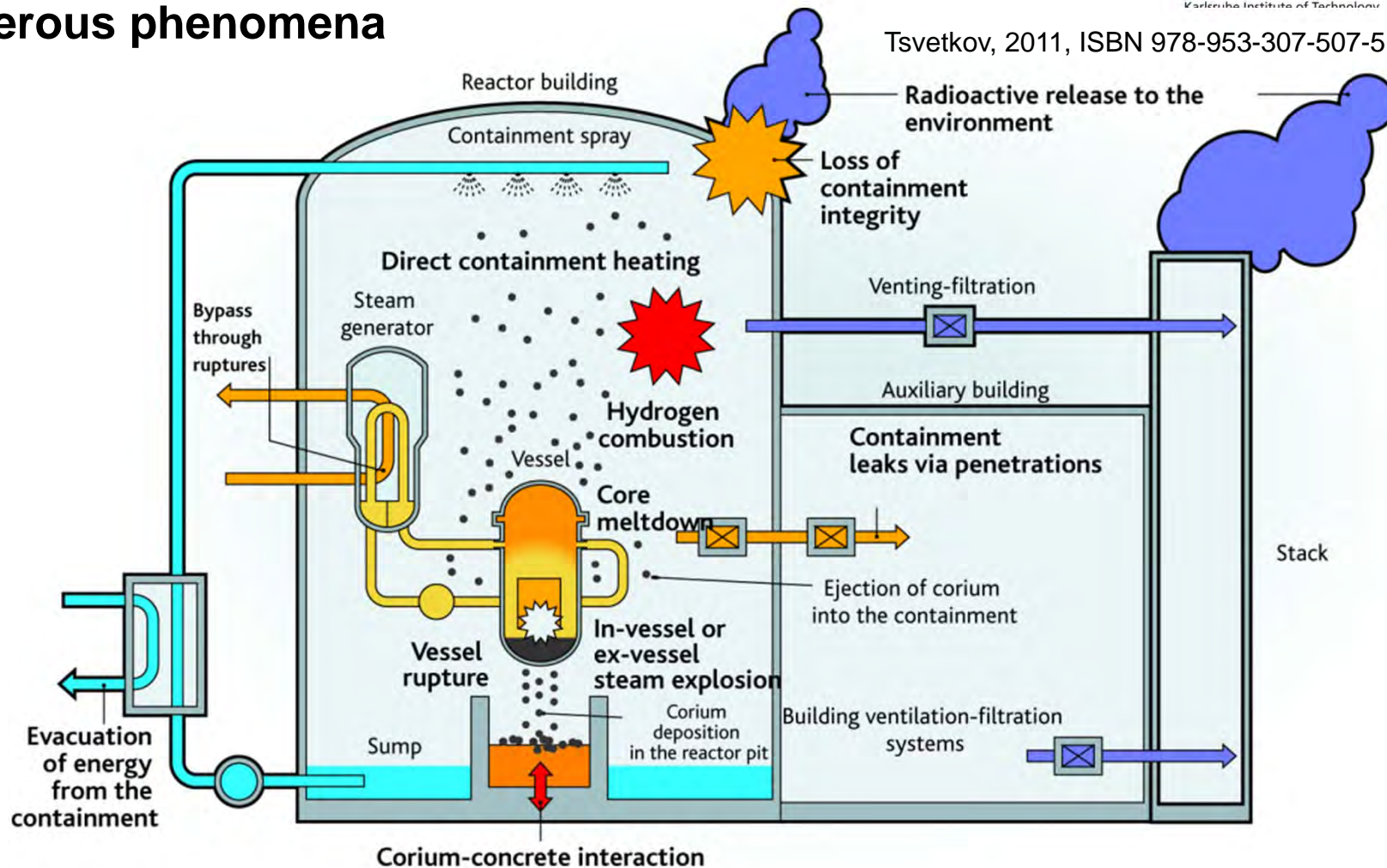
© Westinghouse, AP1000,2012

© AREVA-NP,2011

# Beyond design basis –safety –Severe accidents

## Numerous phenomena

Tsvetkov, 2011, ISBN 978-953-307-507-5



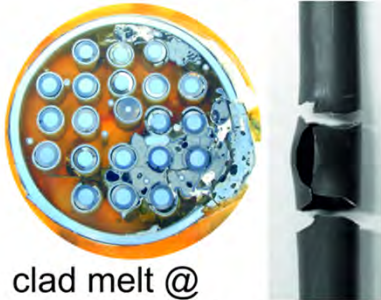
## Subject of international cooperations and networks

Goal: reliable physics description ➔ predictive tool development



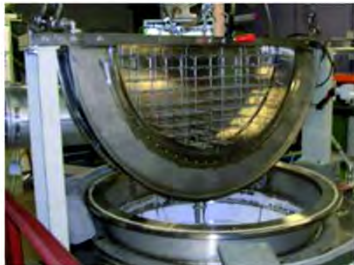
# Beyond design basis –safety –Severe accidents

## Hydrogen generation mechanisms

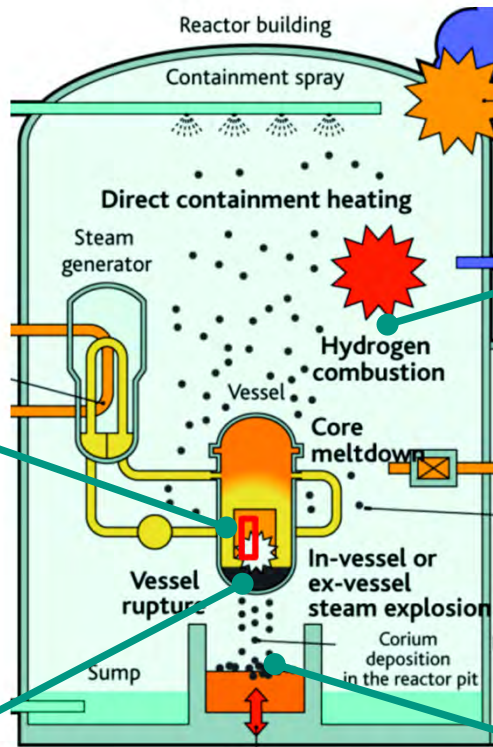


clad melt @  
reflooding hydrogen induced  
clad rupture  
QUENCH prog. @KIT

## Behavior of core melt in lower plenum



LIVE prog. @KIT

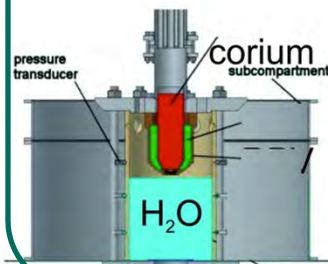


## Hydrogen distribution in large containments



hydrogen safety @KIT

## ex-vessel Molten Core Concrete interaction



- behavior in reactor pit  
- direct containment heating  
DISCO prog. @KIT

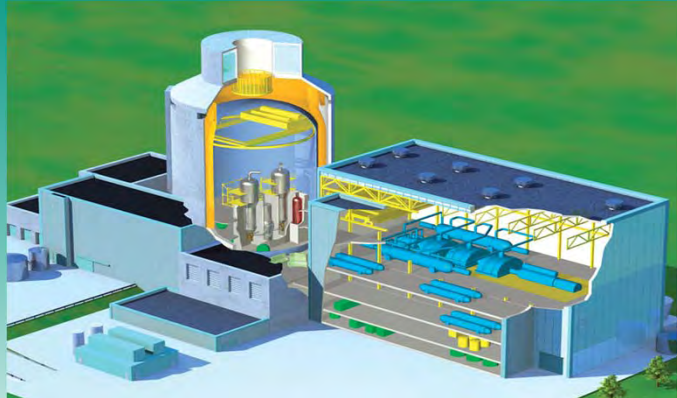


- scale demonstration  
- concrete sensitivity  
MOCKA prog. @KIT

# Large Gen-III Reactors currently deployed (PWR)

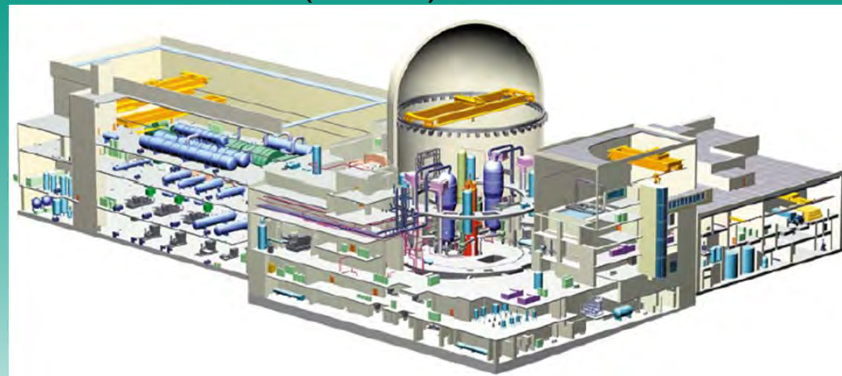


## AP 1000 (Westinghouse –Toshiba)



- 2 SG, 4 Pumps, 1100MWe
- Compact core Passive safety features
- China, US

## APR 1400 (Korea)



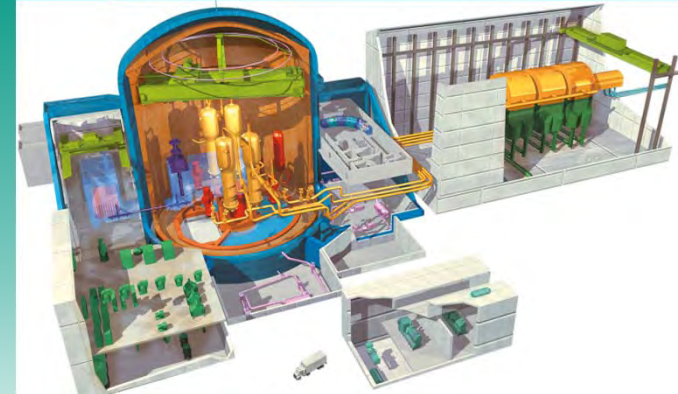
- 2 SG, 4 Pumps, 1400MWe
- 2 act. safety system, no high press. injection
- mixed severe accident strategy
- Korea , UAR

## APWR 1000 (MHI)



- 4 SG, 4 Pumps, 24m fuel cycle,
- 1000MWe
- Instead safety diesels, gas turbine

## EPR (AREVA)



- 4 SG, 4 Pumps, large core , ->1600MWe
- Core catcher, 24m fuel cycle, CDR10<sup>-7</sup>/y
- FIN, FRA, VRC

# Large Gen-III Reactors currently deployed (PWR)

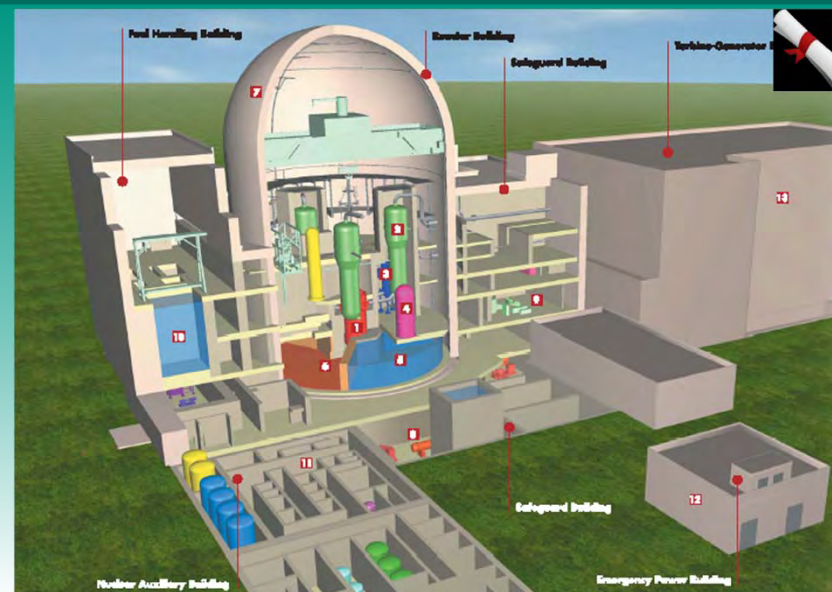
## ❑ AES (Russia)



- 4 SG, 4 Pumps, 1070MWe, Horizontal HEX,
- Passive safety features, Core catcher, soda injection system
- BUL, RUS

## ❑ ATMEA (MHI-AREVA)

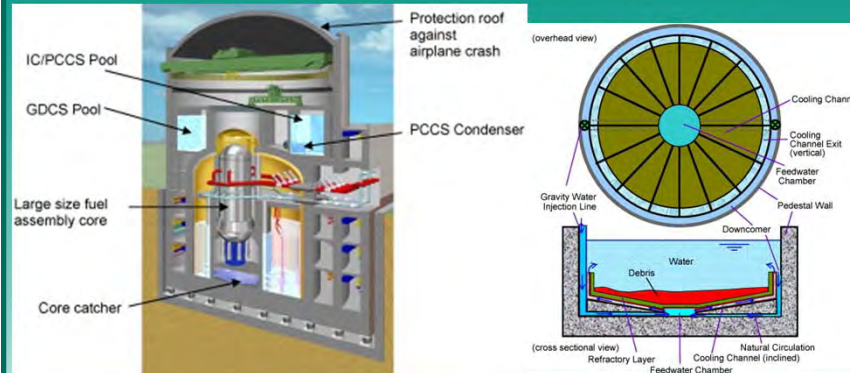
- 3 loop, 1150MWe,
- 3-safety trains
- 2 stage accumulator,
- heavy airplane crash design
- 100% MOX fuelling possible,
- 24m fuel cycle
- interests but no built



# Large Gen-III Reactors currently deployed (BWR)

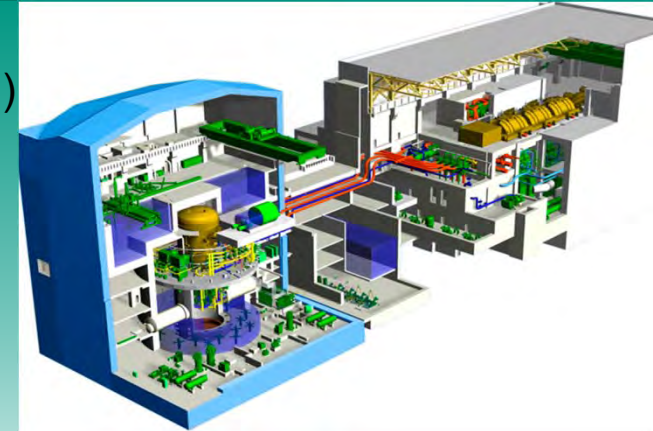


## AB 1600 (Toshiba)



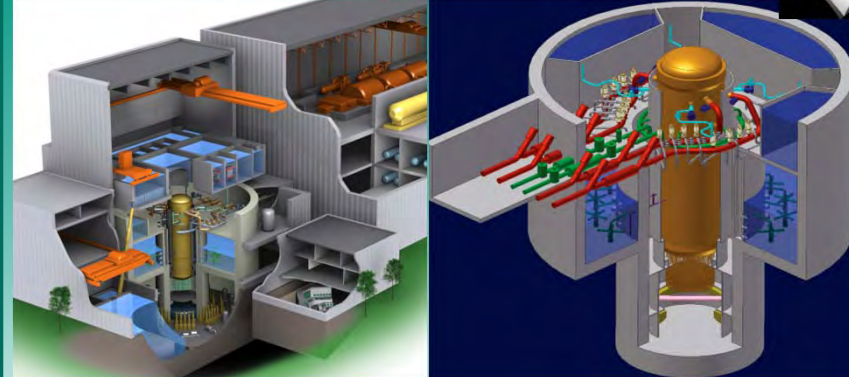
- PCCS (passive containment cooling system)
- GDCS, (gravity based core cooling system).
- Core catcher
- in licensing

## ABWR (Hitachi-GE)



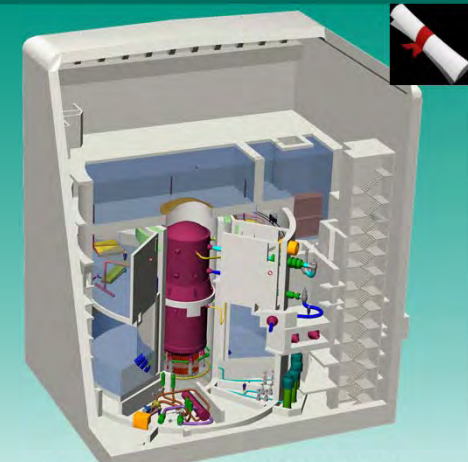
- 1350MWe, high operation flexibility
- high core safety CDR  $<10^{-7}/y$
- short erection time 37m, full MOX capability
- JAP, TAIWAN

## ESBWR (GE)



- 4 passive safety trains (nat. circulation)
- 1500MWe, CDR  $<10^{-8}/y$
- licensed in US, no current projects

## Kerena (AREVA)



- all passive safety system, compact, 1250MWe
- flexible operation, designed for severe acc.
- Airplane crash resistant, no current projects

# SMR operating/ under development (water cooled)

**CAREM-25**  
PWR

- 87MWe
- primary system in vessel

**SMART**  
Korea, Republic of

- 100MWe
- primary system in vessel
- passive DHR

**NuScale**  
PWR

- 45MWe
- nat. circ. cooled
- DHR via containment

**mPower**  
PWR

- 180MWe
- low power density
- 48m fuel cycle
- Passive safety no diesels necess.

**KLT-40s**  
PWR

- 70MWe
- 2 units constructed

**CNP-300**  
PWR

- 300MWe
- 2 loop system
- 3 plant operating
- 2 in construction

**WWER-300**  
PWR

- 300MWe
- In-vessel core catcher

**PHWR-family**  
PHWR

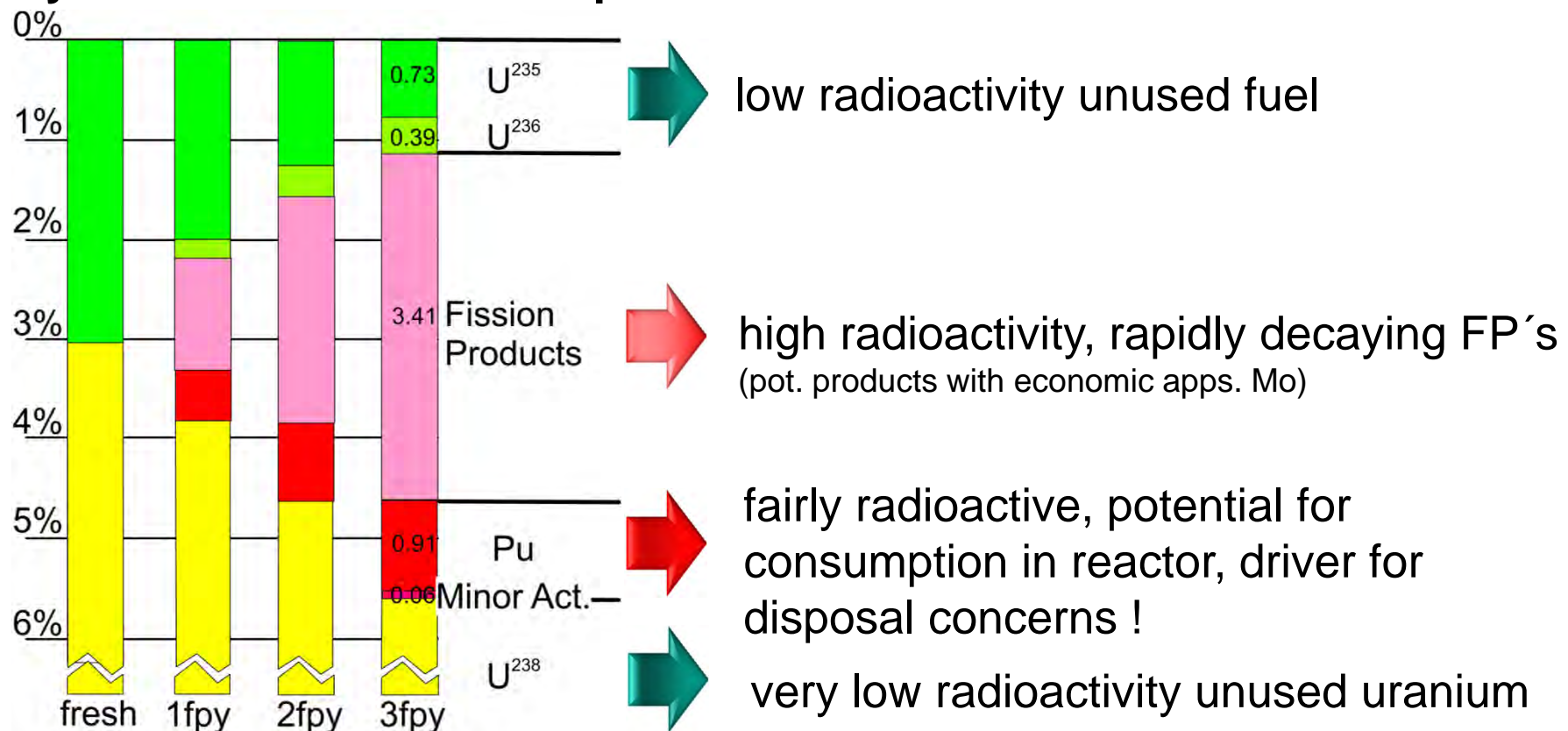
- 220-540MWe
- 2-loop design
- Classic safety des.
- 16 operating plants

# Nuclear Waste

**Nuclear is a generation contract !!!! → requiring acceptance & stability**

- Capital investment
- Long living fission products
- Waste management strategies in all aspects

**Why and what masses to expect ? → Fuel and activated material**



# Nuclear Waste

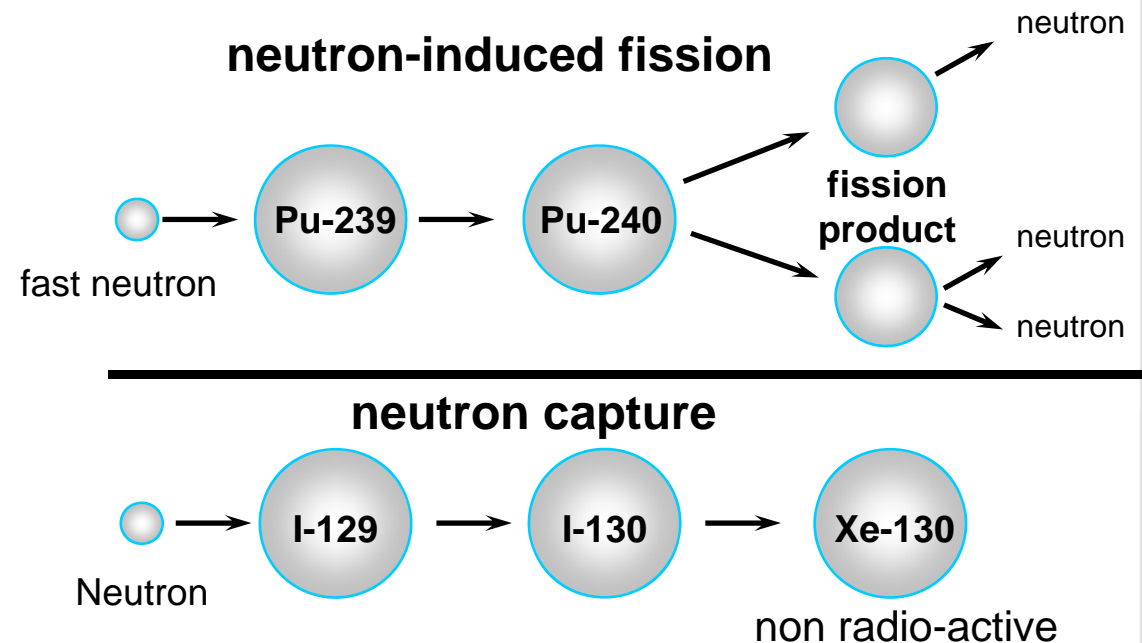
- Reprocessing, conditioning and transport mandatory

## Options for subsequent treatment of radionuclides

- Disposal (geological w/o access, deep underground /near soil ,.....)
- Transmutation

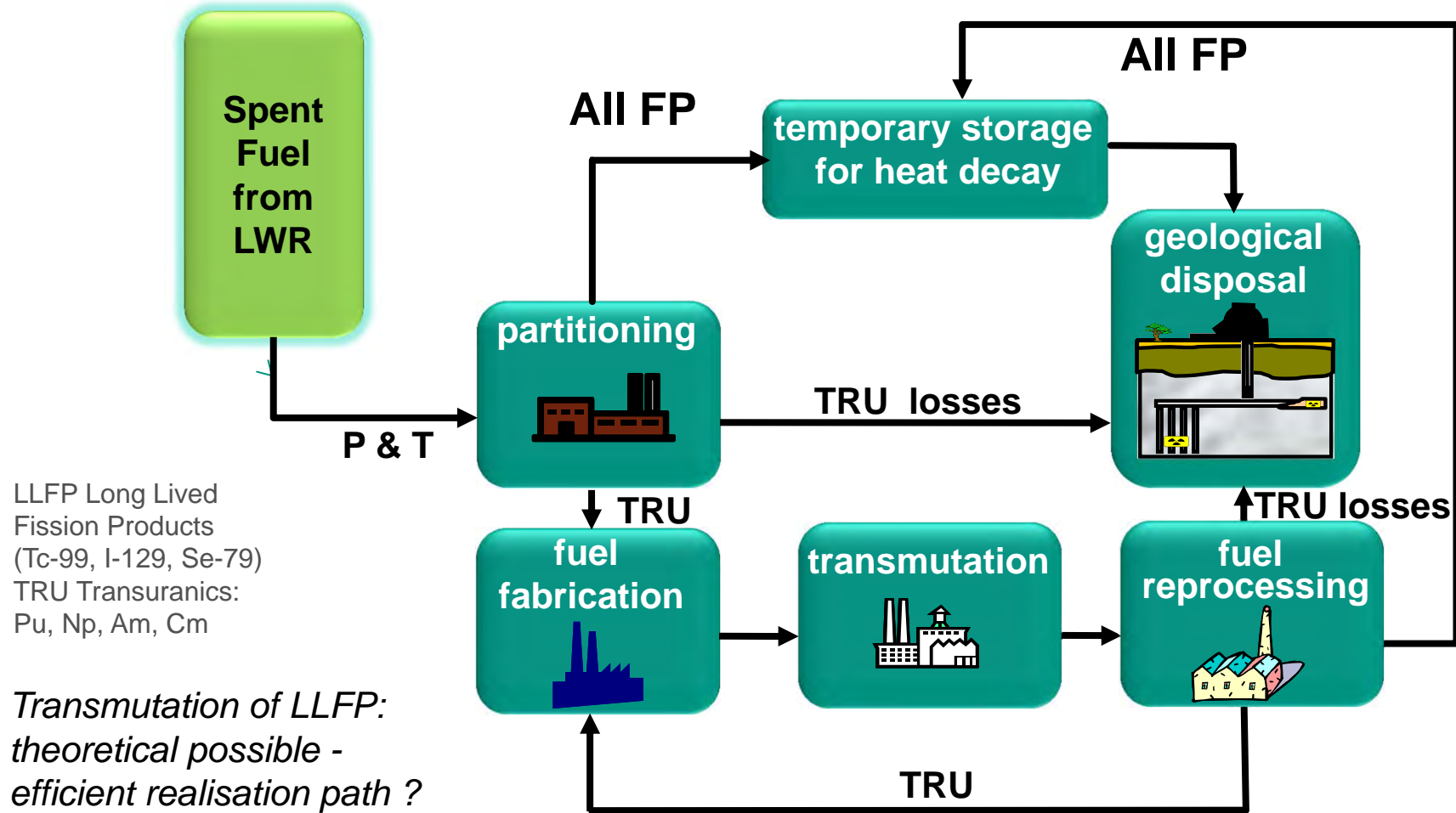
## What is transmutation ?

- transfer of radionuclides by neutron induced fission or neutron capture in another element



# Nuclear Waste -Transmutation

- How to minimize radiologic burdens ? **Fuel cycle required**



➔ **Final repository required but substantially smaller !**

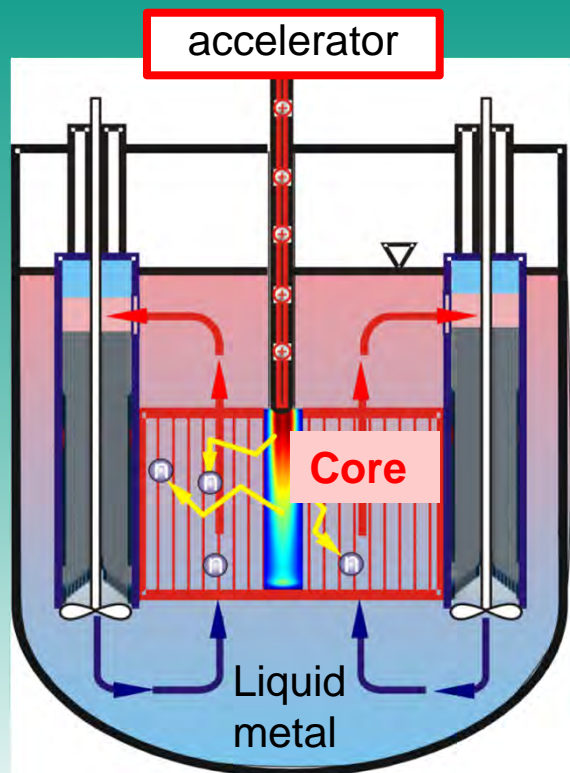


# Nuclear Waste -Transmutation

What type of fast neutron spectrum reactors ? –Two options

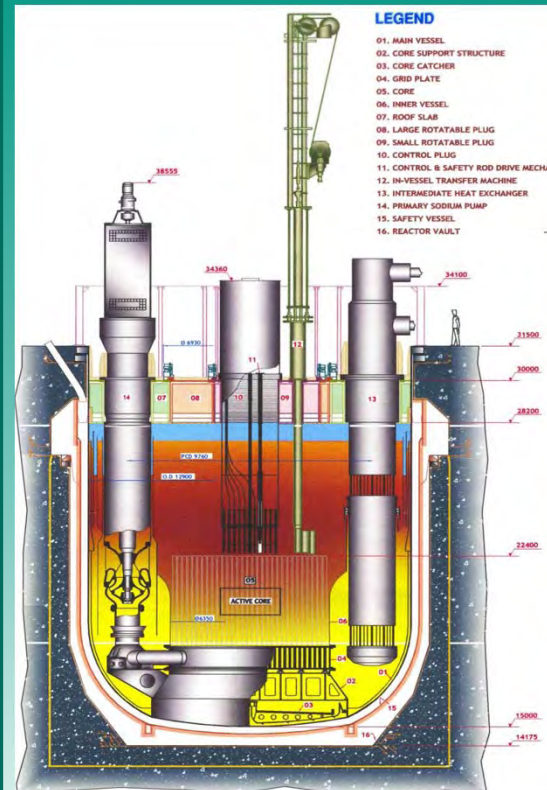
- dependent on further nuclear utilization option !!!

## Accelerator Driven Systems



- Accelerator driven
- Sub-critical core → simply burning

## Fast reactors Gen-IV



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- breeding → fissile regeneration but also
- Burning → transmutation of minor actinides
- critical core - different safety features (!)

# International contributions to Generation IV

## Strategic aims:

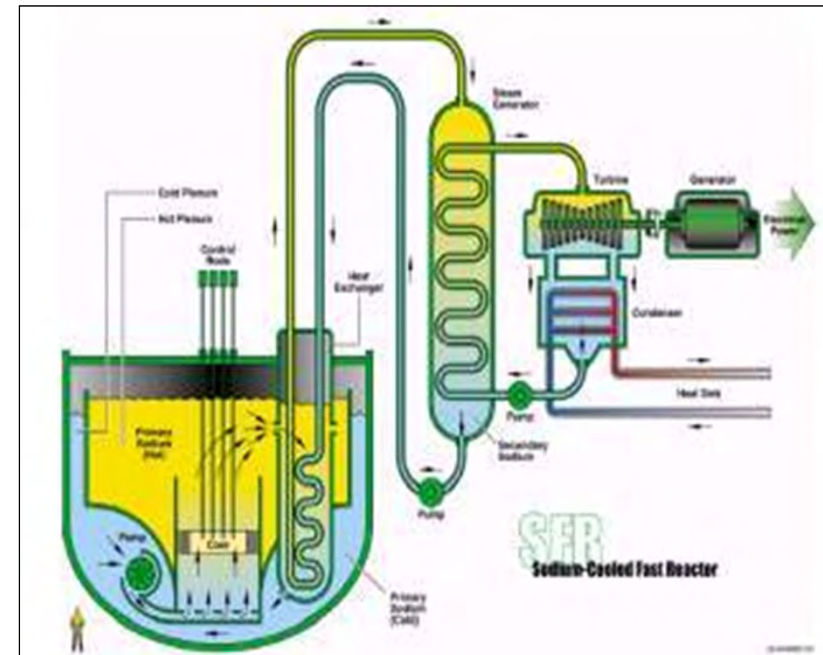
- development of new NPP by 2030 in internat. cooperation
- multifunctionality (electricity, desalination, hydrogen, heat)

## Technologic aims

- better economics
- improved sustainability
- increased safety
- enlarged proliferation resistance

## Status

- continuous worldwide cooperation
- 6 dedicated concepts
- elaboration of standards



U.S.A.



United Kingdom



Switzerland



South Korea



South Africa



Japan



France



Canada



Brazil



Argentina

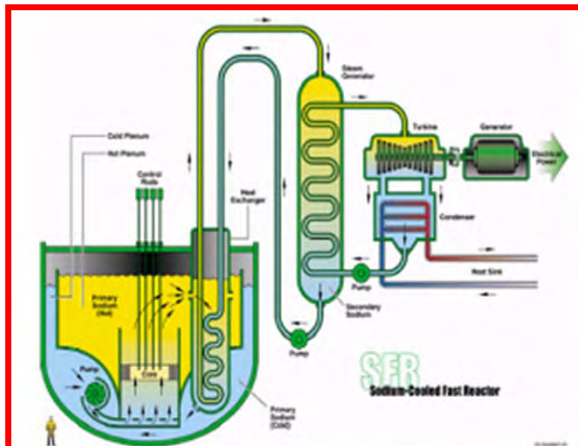


European Union

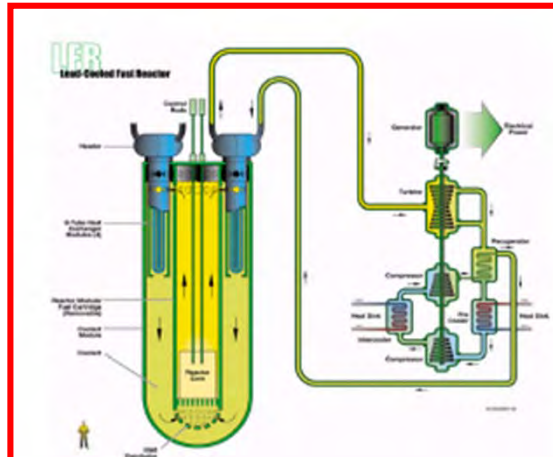
**+China, Russia since 2006!**

**Germany ? –through EU**

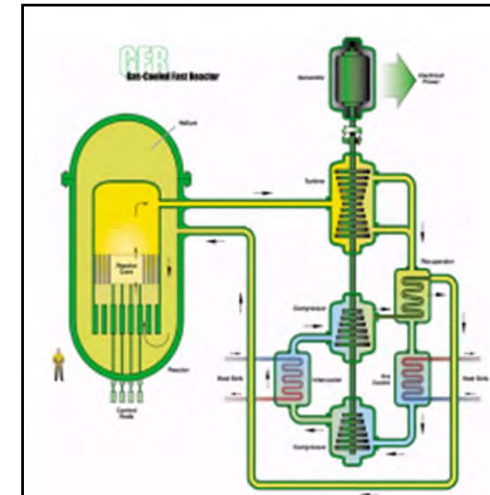
# Generation IV Forum: selection of six nuclear systems



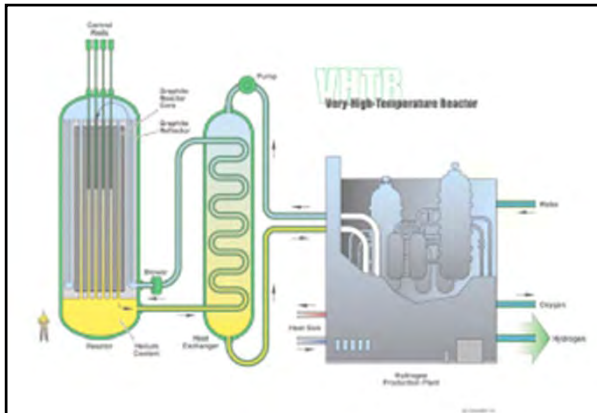
**sodium-cooled fast Reactor**



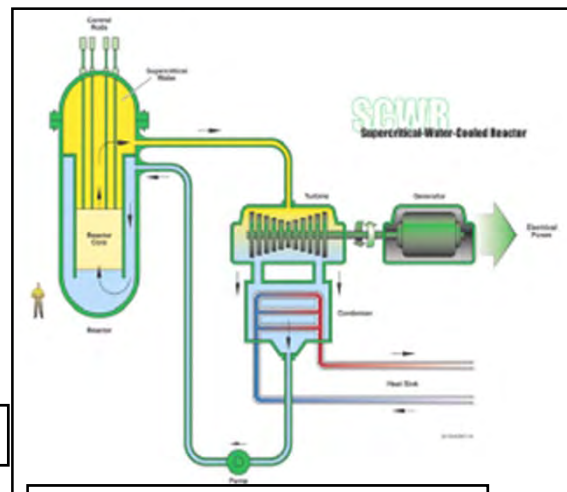
**Lead-cooled Fast Reactor**



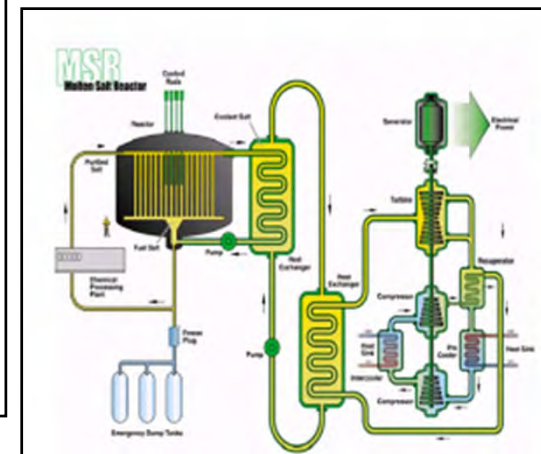
**Gas-cooled Fast Reactor**



**Very High Temperature Reactor**



**Supercritical Water-cooled Reactor**



**Molten Salt Reactor**

# Summary and perspective

- fission energy fission **substantial part** of worldwide energy production.
- mostly generated by Gen –II NPP systems
- **fission pursued worldwide** in numerous industrial countries
- current deployment focused on large scale LWR
- Substantial scientific progress in last decade with respect to safety
  - interesting multi-physics and multi-scale phenomena
  - accurate description of transient processes in plants
  - internationalisation of research and development by collaboration, agreements and bi-lateral contracts
  - current deployment focused on large scale LWR
- nuclear energy production is a **generation contract** !
- nuclear waste management is an essential part of nuclear evolution
- **transmutation** in reactors is a **credible option** to minimize burden on future generations (both: fuel, repository demands)
- irrespective of societal decision on use of nuclear fission energy research, **development and education** must be of vital interest to **assure credible assesement capability**.